

BROWNS FERRY
NUCLEAR PLANT

UNIT 2 CYCLE 10

ASME SECTION XI

NIS-1 DATA REPORT

9908130129 990806
PDR ADOCK 05000260
G PDR



ENCLOSURE 1

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT (BFN)
UNITS 2 AND 3
AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME),
SECTION XI, INSERVICE INSPECTION (ISI), AND
AUGMENTED EXAMINATIONS PROGRAM

SUMMARY REPORT (NIS-1) FOR Unit 2 CYCLE 10 OPERATION
AND CORRECTED SUMMARY REPORT (NIS-1) FOR
UNIT 3 CYCLE 8 OPERATION

(SEE ATTACHED)



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

**INSERVICE INSPECTION
SUMMARY REPORT
UNIT 2 CYCLE 10**



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APPENDIX I

COVER SHEET



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COVER SHEET

DATE OF DOCUMENT COMPLETION

May 9, 1999

NAME OF OWNER AND ADDRESS OF PRINCIPAL OFFICES

Tennessee Valley Authority
Office of Nuclear Power
1101 Market Street
Chattanooga, TN 37402-2801

NAME AND ADDRESS OF THE NUCLEAR GENERATING PLANT

Browns Ferry Nuclear Plant
PO Box 2000
Decatur, AL 35602

NAME OR NUMBER ASSIGNED TO THE NUCLEAR POWER UNIT BY TVA

Browns Ferry Nuclear Plant, Unit 2

COMMERCIAL OPERATION DATE OF UNIT

March 1, 1975



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
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The following information is provided in accordance with the ASME Boiler and Pressure Vessel Code Section XI, 1986 Edition, paragraph IWA-6220.

1. Numbers assigned to the components by the State, Municipality, or Province;
No numbers assigned
2. National Board Numbers assigned to the components by the manufacturer;
No numbers assigned
3. Names of the components and descriptions including size, capacity, material, location, and drawings to aid identification;
The Class 1 and 2 components examined as part of this inservice inspection are listed in Appendix VI, Examination Plan (Section XI).
4. Name and address of manufacturers;
The majority of components examined were supplied by:
General Electric Corporation
San Jose, Ca.
Contract Number: 66C31-90744
5. Manufacturer's component identification numbers;
No numbers assigned
6. Date of completion of the examinations, test, replacement, or repair;
May 9, 1999
7. Name of Inspectors who witnessed or otherwise verified the examinations, tests, replacements, or repairs, and the Inspector's employer and business address;
Mr. Albert L. Ladd and Mr. Bill J. Rice
The Hartford Steam Boiler Inspection and Insurance Company
200 Ashford Center North, Suite 300
Atlanta, GA 30338
8. Abstract of examinations, replacement or repairs performed; conditions recorded; and corrective measures taken;
See Appendices III, IV, VI, and VII
9. Signature of Inspectors;
See Appendix II, Form NIS-1
10. Owner's Report for Inservice Inspections, Form NIS-1;
See Appendix II, Form NIS-1



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APPENDIX II

FORM NIS-1





FORM NIS-1 (back)

- 8. Examination Dates 12/23/98 to 5/09/99
- 9. Inspection Interval from 05/24/92 to 05/24/2001
- 10. Applicable Editions of Section XI 1986 Addenda N/A
- 11. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current interval. See Appendix II, III, IV, V, VI & VII. Examinations complete the first outage of the third period of the second inspection interval.
- 12. Abstract of Conditions Noted See Appendix II, III, IV, V, VI, and VII.
- 13. Abstract of Corrective Measures Recommended and Taken. See Appendix VII

We certify that the statements made in this report are correct and the examinations and corrective measures taken conform to the rules of the ASME Code, Section XI.

Date 7-10 -, 1999 Signed Tennessee Valley Authority
Owner

By [Signature]

Certificate of Authorization No. Not Applicable

Expiration Date Not Applicable

CERTIFICATE OF INSERVICE INSPECTION

I, The undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and/or the State or Province of Tennessee and employed by HSBI & I of Hartford, CT. have inspected the components described in this Owners' Data Report during the period 12/23/98 to 5/09/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owners' Data Report in accordance with the requirements of the ASME Code Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owners' Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspectors Signature

Commissions IBSNA TN 3135
National Board, State, Province and No.

Date July 26 19 99









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EXAMINATION SUMMARY:

The unit 2 cycle 10 Inservice Inspection (ISI) was the first scheduled refueling outage of the third inspection period of the second interval. Approximately 274 visual, 99 ultrasonic, 44 liquid penetrant, and 36 magnetic particle examinations were performed in support of ASME Section XI Code credit, including baseline (PSI), additional samples, and re-examinations (see Appendix VI, Examination Plan). A total of 12 Notification of Indications (NOI's) were issued to document unacceptable conditions identified during the performance of the examinations. These NOI's were evaluated and dispositioned by engineering (see Appendix VII, Summary of Indications).

Additional examinations were performed in accordance with BFN's augmented inspection program and are included for information in Attachment 1. These include ultrasonic inspection of the core shroud access man-way covers, and ultrasonic examination of piping welds for IGSCC. Approximately 48 ultrasonic, 8 magnetic particle, and 3 liquid penetrant examinations were performed in accordance with the augmented program.

Other examinations included the visual examination of the Reactor Pressure Vessel Internals Inspection (RPVII) and the In-Vessel Visual Inspection (IVVI).



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ASME CODE CASES

The following code cases were utilized during the unit 2 cycle 10 outage:

- N-307-1 Revised Ultrasonic Examination Volume for Class 1 Bolting, Table IWB-2500-1, Examination Category B-G-1, When the Examinations Are Conducted From the Center-Drilled Hole, Section XI, Division 1.
- N-435-1 Alternative Examinations Requirements for Vessels With Wall Thickness 2 in. or Less, Section XI, Division 1.
- N-445 Use of Later Editions of SNT-TC-1A for Qualification of Nondestructive Examination Personnel, Section XI, Division 1.
- N-457 Qualification Specimen Notch Location for Ultrasonic Examination of Bolts and Studs, Section XI, Division 1.
- N-460 Alternative Examination Coverage for Class 1 and Class 2 Welds, Section XI, Division 1.
- N-461 Alternative Rules for Piping Calibration Block Thickness, Section XI, Division 1.
- N-491 Alternative Rules for Examination of Class 1, 2, 3, and MC Component Supports of Light Water Cooled Power Plants, Section XI, Division 1.
- N-524 Alternate Examination Requirements for Longitudinal Welds in Class 1 and 2 Piping - Section XI, Division 1.



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UNIT 2 INTERVAL STATUS

The BFN unit 2 cycle 10 ISI examinations were performed during the first scheduled refueling outage of the third period of the second interval. The component quantities examined were selected from 2-SI-4.6.G, Table A (Unit 2 Class 1 and 2 components) and from applicable BFN unit 2 relief requests.

Table 1 summarizes code credit examinations by category and percentage completed and demonstrates compliance with ASME Section XI percentage requirements.

TABLE 1

ASME SECTION XI EXAMINATION SUMMARY FOR THE THIRD PERIOD OF THE SECOND TEN-YEAR INSPECTION INTERVAL

<u>CATEGORY</u>	<u>% COMPLETE</u>	<u>COMMENTS</u>
B-A	57%	
B-B	N/A	
B-D	71%	
B-E	0%	Deferred to system hydro
B-F	62%	
B-G-1	86%	
B-G-2	100%	(Item #'s B7.50 and B7.70 only)
B-G-2	40%	(Item # B7.80 CRD Housing Bolting, inspected when disassembled)
B-H	33%	(100% of OD surface examined, for ID surface reference RFR 2-ISI-1)
B-J	93%	(66% for code credit)
B-K-1	100%	
B-L-1	N/A	
B-L-2	100%	
B-M-1	N/A	
B-M-2	48%	
B-N-1	100%	
B-N-2	70%	Deferral permissible
B-O	0%	Deferral permissible
B-P	-	Refer to pressure test program
B-Q	N/A	
C-A	100%	
C-B	100%	(82% for code credit)
C-C	90%	
C-D	N/A	
C-F-1	61%	
C-F-2	67%	
C-G	N/A	
C-H	-	Refer to pressure test program
D-B	100%	
D-C	100%	
F-A	100%	





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APPENDIX V

METHOD FOR CALCULATION EXAM COVERAGE
EXAMINATIONS.



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METHOD FOR CALCULATION OF LIMITATIONS

The ASME Section XI Code 1989 Edition, no Addenda, requires that "essentially 100%" of the examination area/volume be completed. Code Case N-460 establishes "essentially 100%" to be > 90% coverage; and further states "The applicable examination records shall identify both the cause an percentage of reduced examination coverage". To insure compliance with these requirement during the performance of the ISI examinations the following guidelines are utilized.

The Code required surface examination areas or volumetric examination volumes for each type of weld (piping or vessel weld) are illustrated in the figures of IWB-2500 or IWC-2500.

Surface examination, as illustrated in the figures, are required on essentially 100% of the weld length surface plus a specified amount of base material adjacent to the weld. Surface areas are calculated in those instances where there may be interference and/or limitations causing a reduction in the examination coverage. The Code required surface examination area coverage is generally the weld surface plus 1/2-inch base material adjacent to the weld surface.

Volumetric examinations, as illustrated in the figures, are required for essentially 100% of the specified piping weld volume. The "weld-width" volume includes the weld plus 1/4-inch on each side of the weld, and the "weld-thickness" volume generally includes the lower one-third of the weld/piping thickness for the full length of the weld. As illustrated for vessel welds, the "weld-width" volume generally includes the weld plus 1/2-t (ts/2) on each side of the weld while "weld-thickness" volume generally includes the entire component thickness (i.e. full volume). The volume changes with variations in weld configuration (e.g. transition between different pipe thickness or nozzle-to-vessel configuration). Therefore, it is necessary to determine the required volume for each group of similar welds to allow setting of scanner limits for automated ultrasonic examinations and scan paths and angles for manual ultrasonic examinations.

The 1989 Edition, Section XI, paragraph IWA-2232 requires that the ultrasonic examination of piping systems be conducted in accordance with Appendix I, Section XI. Appendix I requires that the ultrasonic examination of piping systems be conducted in accordance with Appendix III, Section XI and the vessel welds greater than 2-inch thick be conducted in accordance with Article 4 of ASME Section V, 1989 Edition, as supplemented by Appendix I, Section XI. Appendix III and Article 4 define, in part, the applicable examination methods (e.g. examination angles, scan directions) to be used during examination. Paragraphs IWA-2221 and IWA-2222 of Section XI requires that surface examinations be conducted in accordance with Article 6 or 7, as applicable, of ASME Section V, 1989 Edition.

TVA developed procedure N-GP-28 to provide a standardized methodology for calculating percentage of code coverage in those instances where configuration or interference from other components restricts the access for examination and cause an examination limitation/reduction in examination coverage. Components/welds with limitations were evaluated in terms of the feasibility of other NDE techniques or methods to increase coverage or for Code Case N-460 applicability.



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EXAMINATION LIMITATIONS

A tabulation of NDE examination receiving less than the required essentially 100% examination coverage during the unit 2 cycle 10 Inservice Inspection is contained in this Appendix.

The following items/components had examination limitations causing examination coverage to be less than 100%. TVA elected to use Code Case N-460 which states that when the entire examination volume or area cannot be examined due to interference by another component or part geometry, a reduction in examination coverage for Class 1 or Class 2 welds may be accepted provided the reduction in coverage for that weld is less than 10%.

<u>SYSTEM</u>	<u>COMPONENT ID</u>	<u>EXAMINATION METHOD & COVERAGE ACHIEVED</u>	<u>REPORT NO.</u>
RHRS	2-47B452-709-IA	MT > 90%	R-064
RHRS	2-47B452S0274-IA	MT > 90%	R-082
RWCU	RWCU-2-003-026	MT > 90% (PSI EXAM)	R-109
RECIR	2-47B408S0040-IA	PT > 90%	R-177
MSS	2-47B400S0096-IA	PT > 90%	R-272
MSS	3-47B400S0097-IA	PT > 90%	R-273
MSS	2-47B400S0006-IA	PT > 90%	R-318
MSS	2-47B400S0007-IA	PT > 90%	R-321
MSS	2-47B400S0008-IA	PT > 90%	R-329
RPV	N6B-IR	UT > 90%	R-331
RPV	N7-IR	UT > 90%	R-335
RPV	N7-NV	UT > 90%	R-352
RPV	N6B-NV	UT > 90%	R-353
MSS	2-47B400S0005-IA	PT > 90%	R-359

The Reactor Pressure Vessel Support Skirt, Component Number RPV-SUPP-2-1-1A, is accessible for surface examination only from the outside surface. Therefore, a request for relief (RFR) to use Code Case N-323-1 - Alternative Examination for Welded Attachments to Pressure Vessels, is required. The Surveillance Instruction 2-SI-4.6.G, Inservice Inspection Program will be revised to incorporate this examination coverage limitation in the form of RFR.



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APPENDIX VI

EXAMINATION PLAN (SECTION XI)
EXAMINATION PLAN FOR CLASS 1 & CLASS 2 COMPONENTS
COMPONENT ISOMETRIC DRAWINGS



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EXAMINATION PLAN (SECTION XI)

The following printout is an outage ISI report designed to meet the reporting requirements of the ASME Section XI Code, 1986 Edition, Article IWA-6000. The report contains unit 2 cycle 10 ISI data for Class 1 and Class 2 components selected for ASME Section XI credit.

Class 3 data and reports are contained in the Browns Ferry ISI final Report.



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EXAMINATION PLAN OF CLASS 1 & CLASS 2 COMPONENTS

LEGEND FOR EXAMINATION PLAN CODE CREDIT EXAMINATIONS

<u>EXREQ</u>	<u>DEFINITION</u>
86E-02	ASME SECTION XI, 1986 EDITION, SECOND INTERVAL
A01-02	FIRST ADDITIONAL SAMPLE, CATEGORY F-A, PIPING SUPPORTS
P10-02	ASME SECTION XI, PRESERVICE EXAMINATIONS, CYCLE 10, SECOND INTERVAL
R27-02	REPAIR/REPLACEMENT, CATEGORY B-P, ITEM B15.10 LEAKAGE TEST
S01-02	SUCCESSIVE EXAMINATION PERFORMED ON COMPONENTS DUE TO PREVIOUS FINDINGS



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CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
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EXAM REQUIREMENT 86E-02 UNIT: TWO CYCLE: 10 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
CRDS	2-ISV-085-612/3015		R27-02	B-P	B15.70	VT-1		19990502	R-370	P	4 1/2 X 1 1/8 BLT WO98-014041-007
CSS	2-47B458S0004	ISI-0280-C 01	P10-02	F-A	F1.10D	VT-3		19990413	R-115	P	AFS 2.875 WR #98-012557-000
CSS	2-47B458S0013-IA	ISI-0280-C 01	86E-02	B-K-1	B10.10	MT		19990414	R-130	F	ALL INDS. WERE REMOVED PER MMDP-10
CSS	2-47B458S0013-IA	ISI-0280-C 01	86E-02	B-K-1	B10.10	MT		19990420	R-224	P	ALL INDICATIONS WERE REMOVED
CSS	2-CKV-75-0570D		R27-02	B-P	B15.70	VT-1		19990419	R-193	P	REF WO 97-000106-000
CSS	HCV-75-27-BC	ISI-0271-C 01	86E-02	B-G-2	B7.70	VT-1		19990412	R-117	P	
EECW	0-37B205S0059	ISI-0368-C 02	86E-02	F-A	F1.30B	VT-3		19990222	R-004	P	
EECW	1-47B451S0292	ISI-0368-C 10	86E-02	F-A	F1.30B	VT-3		19990223	R-008	P	
EECW	1-47B451S0301	ISI-0368-C 15	86E-02	F-A	F1.30A	VT-3		19990225	R-022	P	
EECW	1-47B451S0435	ISI-0368-C 10	86E-02	F-A	F1.30B	VT-3		19990225	R-021	P	
EECW	1-47B451S0443	ISI-0368-C 13	86E-02	F-A	F1.30A	VT-3		19990225	R-020	P	
EECW	2-47B451R0008	ISI-0368-C 06	86E-02	F-A	F1.30A	VT-3		19990226	R-027	P	
EECW	2-47B451R0031	ISI-0368-C 06	86E-02	F-A	F1.30B	VT-3		19990223	R-011	P	
EECW	2-47B451R0033	ISI-0368-C 06	86E-02	F-A	F1.30D	VT-3		19990223	R-015	P	
EECW	3-17B300S0141	ISI-0368-C 07	86E-02	F-A	F1.30B	VT-3		19990223	R-010	P	
EECW	3-17B300S2003	ISI-0368-C 07	86E-02	F-A	F1.30B	VT-3		19990223	R-009	P	
EECW	3-47B451H0006-IA	ISI-0368-C 09	86E-02	D-B	D2.20	VT-3		19990225	R-019	P	
EECW	3-47B451H0020-IA	ISI-0368-C 08	86E-02	D-B	D2.20	VT-3		19990222	R-007	P	
EECW	3-47B451H0023-IA	ISI-0368-C 08	86E-02	D-B	D2.20	VT-3		19990223	R-017	P	
EECW	3-47B451R0001-IA	ISI-0368-C 09	86E-02	D-B	D2.20	VT-3		19990225	R-025	P	
EECW	3-47B451R0003-IA	ISI-0368-C 09	86E-02	D-B	D2.20	VT-3		19990223	R-012	P	
EECW	3-47B451R0005-IA	ISI-0368-C 09	86E-02	D-B	D2.20	VT-3		19990222	R-005	P	



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EXAM REQUIREMENT 86E-02 UNIT: TWO CYCLE: 10 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
EECW	3-47B451R0015-IA	ISI-0368-C 08	86E-02	D-B	D2.20	VT-3		19990225	R-026	P	SAT.VT-3 PERFORMED; INACCESSIBLE AREAS
EECW	3-47B451R0020-IA	ISI-0368-C 08	86E-02	D-B	D2.20	VT-3		19990223	R-018	P	
EECW	3-47B451R0023	ISI-0368-C 08	86E-02	F-A	F1.30B	VT-3		19990222	R-006	P	
EECW	3-47B451R0028-IA	ISI-0368-C 06	86E-02	D-B	D2.20	VT-3		19990223	R-016	P	
EECW	3-47B451S0025-IA	ISI-0368-C 09	86E-02	D-B	D2.20	VT-3		19990223	R-014	P	
EECW	3-47B451S0061-IA	ISI-0368-C 08	86E-02	D-B	D2.20	VT-3		19990223	R-013	P	
EECW	3-47B451S0296-IA	ISI-0368-C 09	86E-02	D-B	D2.20	VT-3		19990225	R-024	P	
EECW	3-47B451S0310-IA	ISI-0368-C 06	86E-02	D-B	D2.20	VT-3		19990225	R-023	P	
FPCS	2-47B454H0008	ISI-0133-C 01	86E-02	F-A	F1.30A	VT-3		19990225	R-045	P	
FPCS	2-47B454H0008-IA	ISI-0133-C 01	86E-02	D-C	D3.20	VT-3		19990225	R-046	P	
FPCS	2-47B454R0032	ISI-0133-C 01	86E-02	F-A	F1.30B	VT-3		19990308	R-047	P	
FWS	2-47B415H0001-IA	ISI-0277-C 01	86E-02	B-K-1	B10.30	MT		19990423	R-242	P	
FWS	2-47B415H0007-IA	ISI-0277-C 01	86E-02	B-K-1	B10.30	MT		19990423	R-243	P	
FWS	2-47B415S0004	ISI-0277-C 01	P10-02	F-A	F1.10D	VT-3		19990415	R-142	P	AFS 1.05 WR #98-011553-00
FWS	2-47B415S0012	ISI-0277-C 01	P10-02	F-A	F1.10D	VT-3		19990424	R-116	P	AFS 3.75
HPCIS	2-47B455H0058	ISI-0130-C 01	86E-02	F-A	F1.20A	VT-3		19990329	R-100	P	
HPCIS	2-47B455H0067	ISI-0130-C 02	86E-02	F-A	F1.20A	VT-3		19990326	R-097	P	
HPCIS	2-47B455H0074	ISI-0130-C 03	86E-02	F-A	F1.20A	VT-3		19990326	R-087	P	
HPCIS	2-47B455R0020	ISI-0130-C 01	P10-02	F-A	F1.20D	VT-3		19990316	R-066	P	AFS 2.0" WR #98-012567-00
HPCIS	2-47B455S0009	ISI-0130-C 02	86E-02	F-A	F1.20B	VT-3		19990329	R-102	P	
HPCIS	2-47B455S0010	ISI-0130-C 02	86E-02	F-A	F1.20A	VT-3		19990329	R-101	P	
HPCIS	2-RTV-073-0225A		R27-02	B-P	B15.70	VT-1		19990502	R-368	P	4 1/2 X 1 1/8-INCH BLT WO98-014041-005



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
HPCIS	2-SI-3.3.09	N/A	86E-02	C-H	C7.40	VT-2		19990217	R065	P	
HPCIS	2-SI-3.3.09	N/A	86E-02	C-H	C7.60	VT-2		19990217	R065	P	
HPCIS	2-SI-3.3.09	N/A	86E-02	C-H	C7.80	VT-2		19990217	R065	P	
HPCIS	HPCIH-2-1	ISI-0130-C 01	86E-02	F-A	F1.40B	VT-3		19990331	R-106	P	
HPCIS	HPCIH-2-2	ISI-0130-C 02	86E-02	F-A	F1.40B	VT-3		19990330	R-103	P	
HPCIS	HPCIH-2-3	ISI-0130-C 02	86E-02	F-A	F1.40B	VT-3		19990330	R-104	P	
MSS	2-47B2401-35	ISI-0412-C 08	P10-02	F-A	F1.20D	VT-3		19990424	R-312	P	AFS 4* WR#98-013404-000
MSS	2-47B400S0002	ISI-0079-C 01	86E-02	F-A	F1.20B	VT-3		19990424	R-269	P	
MSS	2-47B400S0005	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990501	R-362	P	
MSS	2-47B400S0005-IA	ISI-0279-C 01	86E-02	B-K-1	B10.10	PT		19990429	R-359	P	~94% COVERAGE ACHIEVED
MSS	2-47B400S0006	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990501	R-363	P	
MSS	2-47B400S0006-IA	ISI-0279-C 02	86E-02	B-K-1	B10.10	PT		19990425	R-318	P	~96% COVERAGE ACHIEVED
MSS	2-47B400S0007	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990501	R-364	P	
MSS	2-47B400S0007-IA	ISI-0279-C 02	86E-02	B-K-1	B10.10	PT		19990427	R-321	P	~99% COVEAGE ACHIEVED
MSS	2-47B400S0008	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990430	R-358	P	RIGID SUPPOR; REINSTALLATION
MSS	2-47B400S0008-IA	ISI-0279-C 01	86E-02	B-K-1	B10.10	PT		19990426	R-329	P	
MSS	2-47B400S0096-IA	ISI-0279-C 01	86E-02	B-K-1	B10.10	PT		19990423	R-272	P	~93% COVERAGE OBTAINED, YOKE TO LARGE
MSS	2-47B400S0097	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990422	R-249	P	AFS 3.44* WR#98-011537-000
MSS	2-47B400S0097-IA	ISI-0279-C 01	86E-02	B-K-1	B10.10	PT		19990423	R-273	P	~93% COVERAGE OBTAINED, YOKE TO LARGE
MSS	2-47B400S0103	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990418	R-165	P	AFS 1.75 WR# 98-012655-000
MSS	2-47B400S0103-IA	ISI-0279-C 02	86E-02	B-K-1	B10.10	MT		19990423	R-244	P	
MSS	2-47B400S0104	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990415	R-150	P	AFS 3.375 WR #98-013701-000



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02 UNIT: TWO CYCLE: 10 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
MSS	2-47B400S0105	ISI-0279-C 02	86E-02	F-A	F1.10D	VT-3		19990418	R-186	F	AFS 3.56
MSS	2-47B400S0105	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990422	R-253	P	AFS 4.375 WR#98-013706-000
MSS	2-47B400S0107	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990426	R-330	P	SET 2.875" WR#777
MSS	2-47B400S0108	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990418	R-166	P	AFS 1.125 WR#98-012651-000
MSS	2-47B400S0108-IA	ISI-0279-C 02	86E-02	B-K-1	B10.10	MT		19990423	R-245	P	
MSS	2-47B400S0109	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990425	R-305	P	AFS 1.875" WR#98-012621-000
MSS	2-47B400S0110	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990416	R-149	P	AFS 2.875" WR #98-011689-000
MSS	2-47B400S0212	ISI-0279-C 01	86E-02	F-A	F1.10D	VT-3		19990420	R-198	P	AFS 3.187" & 3.125"
MSS	FCV-01-015	ISI-0222-C 01	P10-02	B-G-2	B7.70	VT-1		19990423	R-265	P	1- BODY STUD & 2 NUTS, WR#97006387-068
MSS	FCV-01-015	ISI-0222-C 01	P10-02	B-M-2	B12.50	VT-3		19990418	R-164	P	BONNET/VALVE S/N 4 WR#98-013625-000
MSS	FCV-01-015-BC	ISI-0222-C 01	P10-02	B-G-2	B7.70	VT-1		19990419	R-196	P	PSI ON ONE NEW STUD
MSS	FCV-01-038	ISI-0222-C 02	P10-02	B-M-2	B12.50	VT-3		19990418	R-163	P	BONNET/VALVE COVER S/N 3
MSS	FCV-01-038	ISI-0222-C 02	86E-02	B-M-2	B12.50	VT-3		19990409	R-113	P	POPPET S/N#4 STEM S/N#1 DISASSEMBLED
MSS	HPAS-2-H-05	ISI-0079-C 02	86E-02	F-A	F1.20C	VT-3		19990425	R-303	P	AFS 578.75
MSS	HPAS-2-H-06	ISI-0079-C 02	86E-02	F-A	F1.20C	VT-3		19990424	R-270	P	AFS 2.0"
MSS	HPAS-2-H-11	ISI-0079-C 02	86E-02	F-A	F1.20C	VT-3		19990424	R-252	F	AFS 2.625, RANGE 3.563 - 4.250
MSS	HPAS-2-H-11	ISI-0079-C 02	P10-02	F-A	F1.20D	VT-3		19990503	R-367	P	AFS 913# WR#99-000239-000
MSS	MS-2-H-14	ISI-0079-C 01	86E-02	F-A	F1.20C	VT-3		19990424	R-271	P	AFS 2.0" & 2.0"
MSS	MSBC-2-08	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990413	R-128	P	
MSS	MSBC-2-09	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-118	P	
MSS	MSBC-2-10	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-119	P	
MSS	MSBC-2-11	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-120	P	



OWNER: TENNESSEE VALLEY AUTHORITY
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 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
MSS	PCV1-2-004	ISI-0312-B 01	P10-02	B-G-2	B7.70	VT-1		19990423	R-266	P	12 NEW STUDS ONLY WO#98-013558-000
MSS	PCV1-2-004	ISI-0312-B 01	86E-02	B-M-2	B12.50	VT-3		19990428	R-343	P	VALVE S/N#1016 WO#98-013558-001
MSS	PCV1-2-004-PBC	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-121	P	
MSS	PCV1-2-005-PBC	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-122	P	
MSS	PCV1-2-018-PBC	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-123	P	
MSS	PCV1-2-019-PBC	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-124	P	
MSS	PCV1-2-022-PBC	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-125	P	
MSS	PCV1-2-023	ISI-0312-B 01	P10-02	B-G-2	B7.70	VT-1		19990423	R-267	P	12NEWSTUDS24NEWNUTS WO#98-013625-000
MSS	PCV1-2-023	ISI-0312-B 01	P10-02	B-M-2	B12.50	VT-3		19990424	R-264	P	WR# 98-013625-000
MSS	PCV1-2-023-PBC	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-126	P	
MSS	PCV1-2-030	ISI-0312-B 01	86E-02	B-G-2	B7.70	VT-1		19981203	R-002	P	
MSS	PCV1-2-030	ISI-0312-B 01	86E-02	B-M-2	B12.50	VT-3		19981203	R-001	P	
MSS	PCV1-2-179-PBC	ISI-0312-B 01	86E-02	B-G-2	B7.50	VT-1		19990412	R-127	P	
RCICS	2-47B456-114-IE	ISI-0131-C 01	86E-02	F-A	F1.40A	VT-3		19990323	R-088	P	
RCICS	2-47B456-116	ISI-0131-C 01	P10-02	F-A	F1.20D	VT-3		19990319	R-072	P	AFS 3.85 WR #98-002774
RCICS	2-47B456H0004	ISI-0131-C 01	86E-02	F-A	F1.20A	VT-3		19990325	R-091	P	
RCICS	2-47B456H0025	ISI-0131-C 01	86E-02	F-A	F1.20A	VT-3		19990323	R-083	P	
RCICS	2-47B456R0001	ISI-0131-C 01	86E-02	F-A	F1.20B	VT-3		19990325	R-090	P	
RCICS	2-47B456R0007	ISI-0131-C 01	P10-02	F-A	F1.20D	VT-3		19990505	R-372	P	AFS 4-3/4" (4" DIA P) WR#98-012564-002
RCICS	2-RTV-071-0001-BC		R27-02	B-P	B15.70	VT-1		19990502	R-369	P	4 1/2 X 1 1/8-INCH BLT WO98-014041-008
RCICS	2-SI-3.3.10	N/A	86E-02	C-H	C7.40	VT-2		19990303	R067	P	
RCICS	2-SI-3.3.10	N/A	86E-02	C-H	C7.40	VT-2		19990303	R068	P	



OWNER: TENNESSEE VALLEY AUTHORITY
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 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RCICS	2-SI-3.3.10	N/A	86E-02	C-H	C7.60	VT-2		19990303	R067	P	
RCICS	2-SI-3.3.10	N/A	86E-02	C-H	C7.80	VT-2		19990303	R067	P	
RCICS	2-SI-3.3.10	N/A	86E-02	C-H	C7.80	VT-2		19990303	R068	P	
RCICS	RCIC-2-009-002	ISI-0129-C 01	P10-02	B-J	B9.11	PT		19990401	R-111	P	
RCICS	RCIC-2-009-003	ISI-0129-C 01	P10-02	B-J	B9.11	PT		19990324	R-092	P	
RCICS	RCIC-2-009-004	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-262	P	
RCICS	RCIC-2-009-009	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-268	P	
RCWS	1-47B446S0053-IA	ISI-0391-C 01	86E-02	D-B	D2.20	VT-3		19990219	R-003	P	
RECIR	2-47B408S0040	ISI-0278-C 02	86E-02	F-A	F1.10C	VT-3		19990417	R-181	P	AFS 10,750# & 10,750#
RECIR	2-47B408S0040-IA	ISI-0278-C 02	86E-02	B-K-1	B10.10	PT		19990417	R-177	P	> 90% COVERAGE
RECIR	2-47B408S0041-IA	ISI-0278-C 02	86E-02	B-K-1	B10.10	PT		19990418	R-173	P	
RECIR	2-47B408S0042	ISI-0278-C 02	A01-02	F-A	F1.10C	VT-3		19990422	R-230	P	AFS 1-13/16"
RECIR	2-47B408S0042-IA	ISI-0278-C 02	86E-02	B-K-1	B10.10	PT		19990418	R-185	P	
RECIR	2-47B408S0043	ISI-0278-C 02	86E-02	F-A	F1.10C	VT-3		19990418	R-174	P	AFS 2.25-INCREMENTS
RECIR	2-47B408S0043-IA	ISI-0278-C 02	86E-02	B-K-1	B10.10	PT		19990418	R-172	P	
RECIR	2-47B408S0045-IA	ISI-0278-C 02	86E-02	B-K-1	B10.10	PT		19990421	R-232	P	
RECIR	2-47B408S0046	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990419	R-187	P	AFS 4.25" WR#98-013724-000
RECIR	2-47B408S0047	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990501	R-361	P	AFS 2-1/2 WR#98-013727-000
RECIR	2-47B408S0047-IA	ISI-0278-C 02	86E-02	B-K-1	B10.20	PT		19990422	R-236	P	
RECIR	2-47B408S0048	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990501	R-360	P	AFS 3-1/2" WR#98-012807-000
RECIR	2-47B408S0053-IE	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990427	R-322	P	AFS 2.562" WR#98-012927-000
RECIR	2-47B408S0054-IE	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990427	R-320	P	AFS 1.312" WR#98-012910-000



OWNER: TENNESSEE VALLEY AUTHORITY
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PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	2-47B408S0055-IA	ISI-0278-C 01	86E-02	B-K-1	B10.10	PT		19990421	R-225	P	
RECIR	2-47B408S0057-IA	ISI-0278-C 01	86E-02	B-K-1	B10.10	PT		19990419	R-183	P	
RECIR	2-47B408S0058	ISI-0278-C 01	P10-02	F-A	F1.10D	VT-3		19990430	R-357	P	AFS 1-3/4" WR#98-012685
RECIR	2-47B408S0058	ISI-0278-C 01	86E-02	F-A	F1.10D	VT-3		19990416	R-151	P	AFS 1.875"
RECIR	2-47B408S0058-IA	ISI-0278-C 01	86E-02	B-K-1	B10.10	PT		19990419	R-184	P	
RECIR	2-47B408S0059	ISI-0278-C 01	A01-02	F-A	F1.10C	VT-3		19990422	R-231	P	AFS 0.9 INCREMENTS
RECIR	2-47B408S0059-IA	ISI-0278-C 01	86E-02	B-K-1	B10.10	PT		19990419	R-182	P	
RECIR	2-47B408S0060	ISI-0278-C 01	A01-02	F-A	F1.10C	VT-3		19990422	R-226	P	AFS 4-3/8 & 4-3/8"
RECIR	2-47B408S0060-IA	ISI-0278-C 01	86E-02	B-K-1	B10.10	PT		19990422	R-229	P	
RECIR	2-47B408S0061	ISI-0278-C 01	86E-02	F-A	F1.40D	VT-3		19990417	R-175	P	AFS 4.34"
RECIR	2-47B408S0061	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990424	R-263	P	AFS 4.5" WR#98-013723-000
RECIR	2-47B408S0062	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990429	R-351	P	AFS 3.75" WR#98-013805-000
RECIR	2-47B408S0062	ISI-0278-C 01	86E-02	F-A	F1.40D	VT-3		19990418	R-171	P	AFS 3.84"
RECIR	2-47B408S0063	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990429	R-336	P	SET 3.75" WR#98-013726-000
RECIR	2-47B408S0063-IA	ISI-0278-C 01	86E-02	B-K-1	B10.20	PT		19990417	R-180	P	
RECIR	2-47B408S0064	ISI-0278-C 01	86E-02	F-A	F1.40C	VT-3		19990418	R-248	P	AFS 3-INCREMENTS & 3.75-INCREMENTS
RECIR	2-47B408S0065	ISI-0278-C 01	86E-02	F-A	F1.40C	VT-3		19990418	R-169	P	AFS 4-INCREMENTS
RECIR	2-47B408S0066	ISI-0278-C 01	86E-02	F-A	F1.40C	VT-3		19990418	R-170	P	PREVIOUSLY REPORTED COND. AFS 3.75 INC
RECIR	2-47B408S0067-IE	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990427	R-338	P	SET 2.6" WR#98-012877-000
RECIR	2-47B408S0068-IE	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990429	R-355	P	AFS 0.375" WR#98-012911-000
RECIR	2-47B408S0069-IE	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990427	R-340	P	AFS 2.25" WR#98-012873-000
RECIR	2-47B408S0070-IA	ISI-0278-C 01	86E-02	B-K-1	B10.10	PT		19990422	R-233	P	



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	2-47B408S0071	ISI-0278-C 01	P10-02	F-A	F1.10D	VT-3		19990425	R-304	P	AFS 4.25" WR#98-01378-000
RECIR	2-47B408S0071	ISI-0278-C 01	A01-02	F-A	F1.10D	VT-3		19990422	R-228	P	AFS 3.5"
RECIR	2-47B408S0071-IA	ISI-0278-C 01	86E-02	B-K-1	B10.10	PT		19990417	R-179	P	
RECIR	2-47B408S0072-IA	ISI-0278-C 01	86E-02	B-K-1	B10.10	PT		19990420	R-199	P	
RECIR	2-47B408S0073	ISI-0278-C 01	P10-02	F-A	F1.10D	VT-3		19990422	R-206	P	AFS 1.75" WR#98-013024-000
RECIR	2-47B408S0074-IA	ISI-0278-C 02	86E-02	B-K-1	B10.10	PT		19990421	R-197	P	
RECIR	2-47B408S0075	ISI-0278-C 02	A01-02	F-A	F1.10	VT-3		19990421	R-218	P	AFS 10810#
RECIR	2-47B408S0076	ISI-0278-C 02	86E-02	F-A	F1.10D	VT-3		19990417	R-162	F	AFS 2.625"
RECIR	2-47B408S0076	ISI-0278-C 02	P10-02	F-A	F1.10D	VT-3		19990428	R-342	P	AFS 2.75 WR#98-013730-000
RECIR	2-47B408S0077	ISI-0278-C 02	A01-02	F-A	F1.10C	VT-3		19990421	R-217	P	AFS 7250#
RECIR	2-47B408S0078	ISI-0278-C 02	A01-02	F-A	F1.10C	VT-3		19990422	R-227	P	AFS 2-11/16"
RECIR	2-47B408S0078	ISI-0278-C 02	P10-02	F-A	F1.10D	VT-3		19990429	R-356	P	AFS 3.5" WR#98-0137731-000
RECIR	2-47B408S0081	ISI-0278-C 01	P10-02	F-A	F1.10D	VT-3		19990428	R-345	P	AFS 2-3/16" WR#98-013824-000
RECIR	2-47B408S0082	ISI-0278-C 02	P10-02	F-A	F1.10D	VT-3		19990415	R-152&1	P	AFS 3.95&1.7 WR #98-013825(28)-000
RECIR	2-47B452S0237	ISI-0278-C 02	86E-02	F-A	F1.10D	VT-3		19990421	R-215	P	AFS 3.125
RECIR	2-47B452S0237	ISI-0278-C 02	P10-02	F-A	F1.10D	VT-3		19990427	R-339	P	AFS 2.375" WR#98-012561-000
RECIR	2-47B452S0237-IA	ISI-0278-C 02	86E-02	B-K-1	B10.10	PT		19990421	R-214	P	
RECIR	PMP-A-NUT-2-01	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-02	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-03	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-04	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-05	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	



OWNER: TENNESSEE VALLEY AUTHORITY
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PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	PMP-A-NUT-2-06	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-07	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-08	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-09	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-10	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-11	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-12	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-13	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-14	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-15	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-NUT-2-16	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT-1		19990421	R-204	P	
RECIR	PMP-A-STUD-2-01	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-02	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990421	R-201	P	
RECIR	PMP-A-STUD-2-03	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-04	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-05	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-06	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-07	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-08	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-09	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-10	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-11	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	



OWNER: TENNESSEE VALLEY AUTHORITY
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PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	PMP-A-STUD-2-12	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-13	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-14	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-15	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-STUD-2-16	ISI-0407-C 01	86E-02	B-G-1	B6.180	UT	BF-119	19990420	R-201	P	
RECIR	PMP-A-WASH-2-01	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-02	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-03	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-04	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-05	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-06	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-07	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-08	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-09	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-10	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-11	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-12	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-13	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-14	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-15	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	PMP-A-WASH-2-16	ISI-0407-C 01	86E-02	B-G-1	B6.200	VT		19990421	R-203	P	
RECIR	RBC-2-2-BC	ISI-0270-C 02	86E-02	B-G-2	B7.50	VT-1		19990421	R-216	P	



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	2-47B452-1206	ISI-0324-C 10	P10-02	F-A	F1.20D	VT-3		19990503	R-366	P	AFS 3-1/8" WEST SNB WR#99-003435-001
RHRS	2-47B452-1320	ISI-0324-C 12	86E-02	F-A	F1.20C	VT-3		19990318	R-076	F	AFS 9119#
RHRS	2-47B452-1320	ISI-0324-C 12	P10-02	F-A	F1.20D	VT-3		19990503	R-365	P	AFS 8366#
RHRS	2-47B452-416	ISI-0324-C 11	P10-02	F-A	F1.20D	VT-3		19990507	R-374	P	AFS 3-1/16" WR#99-003436-034
RHRS	2-47B452-456	ISI-0324-C 09	86E-02	F-A	F1.20A	VT-3		19990322	R-085	P	
RHRS	2-47B452-708-IA	ISI-0324-C 10	86E-02	C-C	C3.20	MT		19990315	R-062	P	REF. EWR99-2-074-013
RHRS	2-47B452-709-IA	ISI-0324-C 10	86E-02	C-C	C3.20	MT		19990315	R-064	P	92% COVERAGE OBTAINED
RHRS	2-47B452-715-IA	ISI-0324-C 11	86E-02	C-C	C3.20	MT		19990319	R-073	P	REF. EWR99-02-074-013
RHRS	2-47B452-717	ISI-0324-C 09	86E-02	F-A	F1.20C	VT-3		19990322	R-084	P	AFS 2,100# & 2,025#
RHRS	2-47B452-722	ISI-0324-C 09	86E-02	F-A	F1.20C	VT-3		19990319	R-077	P	AFS 10,500# & 11,000#
RHRS	2-47B452-968-IA	ISI-0324-C 09	86E-02	C-C	C3.20	MT		19990324	R-081	P	
RHRS	2-47B452-970	ISI-0324-C 08	86E-02	F-A	F1.20D	VT-3		19990324	R-080	P	AFS 3.2"
RHRS	2-47B452-970-IA	ISI-0324-C 08	86E-02	C-C	C3.20	MT		19990325	R-089	P	
RHRS	2-47B452-983-IE	ISI-0324-C 09	86E-02	F-A	F1.40	VT-3		19990322	R-079	P	AFS 85.2# & 86.5#
RHRS	2-47B452-986	ISI-0324-C 10	86E-02	F-A	F1.20B	VT-3		19990322	R-086	P	
RHRS	2-47B452H0085-IA	ISI-0324-C 01	86E-02	C-C	C3.20	MT		19990316	R-060	P	REF. EWR99-2-074-013
RHRS	2-47B452H0091-IA	ISI-0324-C 02	86E-02	C-C	C3.20	MT		19990318	R-074	P	REF. EWR99-2-074-013
RHRS	2-47B452H0119-IA	ISI-0324-C 05	86E-02	C-C	C3.20	MT		19990312	R-058	P	REF. EWR99-2-074-013
RHRS	2-47B452H0121-IA	ISI-0324-C 05	86E-02	C-C	C3.20	MT		19990312	R-057	P	REF. EWR99-2-074-013
RHRS	2-47B452H0122-IA	ISI-0324-C 05	86E-02	C-C	C3.20	MT		19990316	R-061	P	REF. EWR99-2-074-013
RHRS	2-47B452H0124-IA	ISI-0324-C 07	86E-02	C-C	C3.20	MT		19990317	R-070	P	REF. EWR99-2-074-013
RHRS	2-47B452H0126-IA	ISI-0324-C 07	86E-02	C-C	C3.20	MT		19990317	R-071	P	REF. EWR99-02-074-013



OWNER: TENNESSEE VALLEY AUTHORITY
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 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	2-47B452H0128-IA	ISI-0324-C 07	86E-02	C-C	C3.20	MT		19990318	R-075	P	REF. EWR99-2-074-013
RHRS	2-47B452R0051	ISI-0324-C 04	P10-02	F-A	F1.20D	VT-3		19990505	R-373	P	AFS 2.95" WR#99-003438-017
RHRS	2-47B452S0113	ISI-0276-C 01	86E-02	F-A	F1.10B	VT-3		19990418	R-167	F	BROKEN FILLET/TACK WELDS
RHRS	2-47B452S0152-IA	ISI-0324-C 05	86E-02	C-C	C3.20	MT		19990312	R-056	P	
RHRS	2-47B452S0227	ISI-0276-C 01	P10-02	F-A	F1.10D	VT-3		19990423	R-247	P	AFS 1.375" WR#98-013698-000
RHRS	2-47B452S0227	ISI-0276-C 01	86E-02	F-A	F1.10D	VT-3		19990414	R-134	P	AFS 2.125"
RHRS	2-47B452S0229	ISI-0276-C 01	86E-02	F-A	F1.10C	VT-3		19990415	R-133	F	AFS 9,119
RHRS	2-47B452S0229	ISI-0276-C 01	86E-02	F-A	F1.10C	VT-3		19990419	R-192	P	AFS 9,119
RHRS	2-47B452S0234	ISI-0276-C 01	86E-02	F-A	F1.10C	VT-3		19990414	R-132	F	AFS 10,622#
RHRS	2-47B452S0234	ISI-0276-C 01	86E-02	F-A	F1.10C	VT-3		19990419	R-195	P	RE-EXAM FOR MISSING NUT REPLACEMENT
RHRS	2-47B452S0235	ISI-0276-C 01	P10-02	F-A	F1.10D	VT-3		19990428	R-341	P	AFS 2.43" WR#98-012563-000
RHRS	2-47B452S0240	ISI-0276-C 01	P10-02	F-A	F1.10D	VT-3		19990423	R-246	P	AFS 1.437" WR#98-013696-000
RHRS	2-47B452S0241-IA	ISI-0324-C 05	86E-02	C-C	C3.20	MT		19990315	R-063	P	REF. EWR99-2-074-013
RHRS	2-47B452S0250	ISI-0324-C 09	P10-02	F-A	F1.20D	VT-3		19990330	R-099	P	AFS 3.9" WR #98-002770
RHRS	2-47B452S0251	ISI-0324-C 09	P10-02	F-A	F1.20D	VT-3		19990509	R-375	P	AFS 4.5/8" WR#99-003437-001
RHRS	2-47B452S0251-IA	ISI-0324-C 09	86E-02	C-C	C3.20	MT		19990322	R-078	P	
RHRS	2-47B452S0274-IA	ISI-0324-C 09	86E-02	C-C	C3.20	MT		19990324	R-082	P	>90% COVERAGE
RHRS	2-SHV-074-0754		R27-02	B-P	B15.70	VT-1		19990502	R-371	P	NEW BOLTING WO 98-014041-004
RHRS	FCV-74-68-BC	ISI-0221-C 01	86E-02	B-G-2	B7.70	VT-1		19990414	R-138	P	
RHRS	RHRG-2-06A-A	ISI-0406-C 01	86E-02	C-B	C2.31	MT		19990311	R-052	P	
RHRS	RHRG-2-06B-A	ISI-0406-C 01	86E-02	C-B	C2.31	MT		19990311	R-051	P	
RHRS	RHRG-2-08-A	ISI-0406-C 01	86E-02	C-A	C1.10	UT	BF-40	19990310	R-055	P	100% COVERAGE



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PLANT: BROWNS FERRY NUCLEAR PLANT
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 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	RHRG-2-10-A	ISI-0406-C 01	86E-02	C-A	C1.20	UT	BF-40	19990310	R-053	P	100% COVERAGE
RHRS	RHRG-2-12-A	ISI-0406-C 01	S01-02	F-A	F1.40B	VT-3		19990311	R-050	P	SUCCESSIVE EXAM
RHRS	RHRG-2-12-C	ISI-0406-C 01	S01-02	F-A	F1.40B	VT-3		19990311	R-049	P	SUCCESSIVE EXAM
RHRS	RHRG-2-12-C-IA	ISI-0406-C 01	86E-02	C-C	C3.10	MT		19990312	R-054	P	
RHRS	RHRG-2-13-A	ISI-0406-C 01	S01-02	F-A	F1.40B	VT-3		19990311	R-048	F	SUCCESSIVE EXAM
RHRS	RHRG-2-14-D	ISI-0406-C 01	S01-02	F-A	F1.40B	VT-3		19990317	R-069	F	SUCCESSIVE EXAM
RHRS	2-47B450H0033-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990302	R-031	P	
RHRS	2-47B450H0034-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990301	R-030	P	
RHRS	2-47B450H0035-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990301	R-029	P	
RHRS	2-47B450H0036-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990302	R-028	P	
RHRS	2-47B450H0040	ISI-0145-C 02	86E-02	F-A	F1.30C	VT-3		19990304	R-041	P	
RHRS	2-47B450H0040-IA	ISI-0145-C 02	86E-02	D-B	D2.40	VT-3		19990304	R-044	P	
RHRS	2-47B450H0041-IA	ISI-0145-C 02	86E-02	D-B	D2.40	VT-3		19990304	R-040	P	
RHRS	2-47B450H0042	ISI-0145-C 02	86E-02	F-A	F1.30C	VT-3		19990304	R-037	F	
RHRS	2-47B450H0044-IA	ISI-0145-C 03	86E-02	D-B	D2.20	VT-3		19990303	R-035	P	
RHRS	2-47B450H0049-IA	ISI-0145-C 03	86E-02	D-B	D2.20	VT-3		19990304	R-042	P	
RHRS	2-47B450R0023-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990301	R-034	P	
RHRS	2-47B450R0024-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990301	R-032	P	
RHRS	2-47B450R0027-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990304	R-038	P	
RHRS	2-47B450R0029-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990301	R-033	P	
RHRS	2-47B450R0030-IA	ISI-0145-C 02	86E-02	D-B	D2.20	VT-3		19990304	R-039	P	
RHRS	2-47B450R0035-IA	ISI-0145-C 03	86E-02	D-B	D2.20	VT-3		19990226	R-043	P	



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02 UNIT: TWO CYCLE: 10 COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	2-47B450R0040-IA	ISI-0145-C 03	86E-02	D-B	D2.20	VT-3		19990303	R-036	P	
RPV	2-SI-3.3.01A	N/A 01	86E-02	B-P	B15.10	VT-2		19990502	R-376	P	
RPV	2-SI-3.3.01A	N/A 01	86E-02	B-P	B15.50	VT-2		19990502	R-376	P	
RPV	2-SI-3.3.01A	N/A 01	86E-02	B-P	B15.60	VT-2		19990502	R-376	P	
RPV	2-SI-3.3.01A	N/A 01	86E-02	B-P	B15.70	VT-2		19990502	R-376	P	
RPV	CORE-SUP-STR	N/A 00	86E-02	B-N-2	B13.40	VT-3		19990424	R-251	P	
RPV	CRDN-2-0631-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990416	R-288	P	CRD BOLTS & WASHERS
RPV	CRDN-2-1019-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990417	R-294	P	CRD BOLTS & WASHERS
RPV	CRDN-2-1051-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990416	R-289	P	CRD BOLTS & WASHERS
RPV	CRDN-2-1427-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990417	R-295	P	CRD BOLTS & WASHERS
RPV	CRDN-2-1435-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990416	R-290	P	CRD BOLTS & WASHERS
RPV	CRDN-2-1827-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990417	R-296	P	CRD BOLTS & WASHERS
RPV	CRDN-2-1831-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990417	R-297	P	CRD BOLTS & WASHERS
RPV	CRDN-2-1843-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990417	R-298	P	CRD BOLTS & WASHERS
RPV	CRDN-2-2231-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990417	R-299	P	CRD BOLTS & WASHERS
RPV	CRDN-2-2247-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990416	R-291	P	CRD BOLTS & WASHERS
RPV	CRDN-2-2619-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990417	R-300	P	CRD BOLTS & WASHERS
RPV	CRDN-2-2655-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990416	R-292	P	CRD BOLTS & WASHERS
RPV	CRDN-2-3043-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990417	R-301	P	CRD BOLTS & WASHERS
RPV	CRDN-2-3047-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990414	R-275	P	CRD BOLTS & WASHERS
RPV	CRDN-2-3403-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990415	R-280	P	CRD BOLTS & WASHERS
RPV	CRDN-2-3411-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990415	R-281	P	CRD BOLTS & WASHERS



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PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	CRDN-2-3443-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990416	R-293	P	CRD BOLTS & WASHERS
RPV	CRDN-2-3803-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990415	R-282	P	CRD BOLTS & WASHERS
RPV	CRDN-2-3819-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990415	R-283	P	CRD BOLTS & WASHERS
RPV	CRDN-2-3851-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990414	R-276	P	CRD BOLTS & WASHERS
RPV	CRDN-2-4259-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990413	R-274	P	CRD BOLTS & WASHERS
RPV	CRDN-2-4635-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990416	R-285	P	CRD BOLTS & WASHERS
RPV	CRDN-2-4655-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990419	R-302	P	CRD BOLTS & WASHERS
RPV	CRDN-2-5027-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990416	R-286	P	CRD BOLTS & WASHERS
RPV	CRDN-2-5043-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990415	R-287	P	CRD BOLTS & WASHERS
RPV	CRDN-2-5415-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990414	R-277	P	CRD BOLTS & WASHERS
RPV	CRDN-2-5435-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	PT		19990414	R-278	P	PT CRACKLIKE IND.
RPV	CRDN-2-5435-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990414	R-279	P	CRD BOLTS & WASHERS; NEW BOLT
RPV	CRDN-2-5843-BC	ISI-0292-C 01	86E-02	B-G-2	B7.80	VT-1		19990415	R-284	P	CRD BOLTS & WASHERS
RPV	N6B-2-2-BC	ISI-0408-C 01	86E-02	B-G-2	B7.50	VT-1		19990416	R-176	P	
RPV	N6B-IR	ISI-0408-C 01	86E-02	B-D	B3.100	UT	BF-81	19990425	R-331	P	>90% COVERAGE ACHIEVED; SPECIAL SHOES
RPV	N6B-NV	ISI-0408-C 01	86E-02	B-D	B3.90	UT	BF-19	19990425	R-353	P	>90% COVERAGE ACHIEVED
RPV	N7-2-1	ISI-0408-C 01	86E-02	B-J	B9.11	MT		19990416	R-160	P	
RPV	N7-2-1	ISI-0408-C 01	86E-02	B-J	B9.11	UT	BF-30	19990418	R-178	P	
RPV	N7-2-3-BC	ISI-0408-C 01	86E-02	B-G-2	B7.50	VT-1		19990416	R-194	P	DISASSEMBLED
RPV	N7-IR	ISI-0408-C 01	86E-02	B-D	B3.100	UT	BF-81	19990425	R-335	P	>90% COVERAGE ACHIEVED; SPECIAL SHOES
RPV	N7-NV	ISI-0408-C 01	86E-02	B-D	B3.90	UT	BF-19	19990423	R-352	P	>90% COVERAGE ACHIEVED
RPV	RCH-2-2C	ISI-0408-C 01	86E-02	B-A	B1.40	UT	BF-19	19990423	R-337	P	EXAM PERFORMED FROM INSIDE SURFACE



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 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RCH-2-2C-FLEX	ISI-0408-C 01	86E-02	B-A	B1.40	MT		19990415	R-139	P	PERFORMED ON OUTSIDE SURFACE
RPV	RPV INT ATT BLR	CHM-2046-C 02	86E-02	B-N-2	B13.20	VT-3		19990424	R-251	P	SEE REPORT FOR ITEMS EXAMINED
RPV	RPV INT ATT NBLR	CHM-2046-C 02	86E-02	B-N-2	B13.30	VT-3		19990424	R-251	P	SEE REPORT FOR ITEMS EXAMINED
RPV	RPV-BUSH-2-32	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-33	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-34	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-61	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-62	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-63	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-64	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-65	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-66	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-67	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-68	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-69	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-70	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-71	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-72	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-73	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-74	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-75	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-76	ISI-0266-C 01	86E-02	B-G-1	B6.50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE



OWNER: TENNESSEE VALLEY AUTHORITY
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 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-BUSH-2-77	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-78	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-79	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-80	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-81	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-82	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-83	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-84	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-85	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-86	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-87	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-88	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-89	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-90	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-91	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-BUSH-2-92	ISI-0266-C 01	86E-02	B-G-1	B6. 50	VT-1		19990413	R-131	P	STUDS WERE IN PLACE
RPV	RPV-INTERIOR	CHM-2046-C 02	86E-02	B-N-1	B13.10	VT-3		19990424	R-251	P	SEE REPORT FOR ITEMS EXAMINED
RPV	RPV-LIGS-2-01	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990412	R-147	P	
RPV	RPV-LIGS-2-02	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-03	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-04	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-05	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	



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 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-LIGS-2-06	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-07	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-34	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-35	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-36	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-37	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-69	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990414	R-147	P	
RPV	RPV-LIGS-2-70	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-71	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-72	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-73	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-74	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-75	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-76	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-77	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-78	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-79	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-80	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-81	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-82	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-83	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-84	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-LIGS-2-85	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-86	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-87	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-88	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-89	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-90	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-91	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-LIGS-2-92	ISI-0266-C 01	86E-02	B-G-1	B6. 40	UT	BF-126	19990413	R-147	P	
RPV	RPV-STUDS-2-09	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-11	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-13	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-15	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-17	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-18	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-19	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-21	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-22	ISI-0266-C 01	86E-02	B-G-1	B6. 30	MT		19990415	R-135	P	
RPV	RPV-STUDS-2-22	ISI-0266-C 01	86E-02	B-G-1	B6. 30	UT	BF-126	19990414	R-143	P	REMOVED
RPV	RPV-STUDS-2-23	ISI-0266-C 01	86E-02	B-G-1	B6. 30	MT		19990415	R-135	P	
RPV	RPV-STUDS-2-23	ISI-0266-C 01	86E-02	B-G-1	B6. 30	UT	BF-126	19990414	R-143	P	REMOVED
RPV	RPV-STUDS-2-24	ISI-0266-C 01	86E-02	B-G-1	B6. 30	MT		19990415	R-135	P	
RPV	RPV-STUDS-2-24	ISI-0266-C 01	86E-02	B-G-1	B6. 30	UT	BF-126	19990414	R-143	P	REMOVED



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-STUDS-2-25	ISI-0266-C 01	86E-02	B-G-1	B6. 30	MT		19990415	R-135	P	
RPV	RPV-STUDS-2-25	ISI-0266-C 01	86E-02	B-G-1	B6. 30	UT	BF-126	19990414	R-143	P	REMOVED
RPV	RPV-STUDS-2-27	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-29	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-31	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-33	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-35	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-37	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-39	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-41	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-42	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-45	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-47	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-48	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-49	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-50	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-51	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-53	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-55	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-57	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-59	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-72	ISI-0266-C 01	86E-02	B-G-1	B6. 20	UT	BF-126	19990412	R-143	P	INPLACE



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	RPV-STUDS-2-90	ISI-0266-C 01	86E-02	B-G-1	B6.20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-91	ISI-0266-C 01	86E-02	B-G-1	B6.20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-STUDS-2-92	ISI-0266-C 01	86E-02	B-G-1	B6.20	UT	BF-126	19990412	R-143	P	INPLACE
RPV	RPV-SUPP-2-1	ISI-0415-C 02	86E-02	F-A	F1.40B	VT-3		19990421	R-207	P	RUST, NO WALL LOSS
RPV	RPV-SUPP-2-1-IA	ISI-0415-C 02	86E-02	B-H	B8.10	MT		19990420	R-205	P	RFR FOR USE OF CC N323-1;
RWCU	2-47B406S0018	ISI-0274-C 01	86E-02	F-A	F1.10C	VT-3		19990429	R-200	P	
RWCU	2-47B406S0020	ISI-0274-C 01	86E-02	F-A	F1.10B	VT-3		19990426	R-319	P	
RWCU	FCV-69-001	ISI-0272-C 01	P10-02	B-M-2	B12.50	VT-3		19990406	R-107	P	ACCESSABLE INTERNAL SURFACES
RWCU	FCV-69-002	ISI-0272-C 01	P10-02	B-M-2	B12.50	VT-3		19990406	R-108	P	
RWCU	RWCU-2-003-025	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990422	R-223	P	
RWCU	RWCU-2-003-025	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-306	P	AFTER RT REPAIR
RWCU	RWCU-2-003-025	ISI-0272-C 01	P10-02	B-J	B9.11	UT	BF-61	19990425	R-315	P	100% COVERAGE NEW WELD
RWCU	RWCU-2-003-026	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990331	R-112	P	
RWCU	RWCU-2-003-026	ISI-0272-C 01	P10-02	B-J	B9.11	UT		19990405	R-109	P	
RWCU	RWCU-2-003-027	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-307	P	AFTER REPAIR
RWCU	RWCU-2-003-027	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990426	R-314	P	AFTER REPAIR
RWCU	RWCU-2-003-027	ISI-0272-C 01	P10-02	B-J	B9.11	UT	BF-61	19990426	R-316	P	100% COVERAGE NEW WELD
RWCU	RWCU-2-003-037	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990428	R-346	P	
RWCU	RWCU-2-003-038	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990428	R-346	P	
RWCU	RWCU-2-003-039	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990315	R-347	P	DUPLICATE OF REPORT R-094
RWCU	RWCU-2-003-039	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-094	P	
RWCU	RWCU-2-003-040	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-095	P	



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

EXAM REQUIREMENT 86E-02

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

System	Component Number	ISO Drawing	Exreq	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RWCU	RWCU-2-003-040	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-348	P	DUPLICATE OF REPORT R-095
RWCU	RWCU-2-003-041	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990428	R-346	P	
RWCU	RWCU-2-003-042	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-096	P	
RWCU	RWCU-2-003-042	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-349	P	DUPLICATE OF REPORT R-096
RWCU	RWCU-2-003-043	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-093	P	
RWCU	RWCU-2-003-043	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-350	P	DUPLICATE OF REPORT R-093
RWCU	RWCU-2-003-044	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-308	P	
RWCU	RWCU-2-003-044	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990426	R-313	P	
RWCU	RWCU-2-003-044	ISI-0272-C 01	P10-02	B-J	B9.11	UT	BF-61	19990426	R-317	P	100% COVERAGE NEW WELD



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

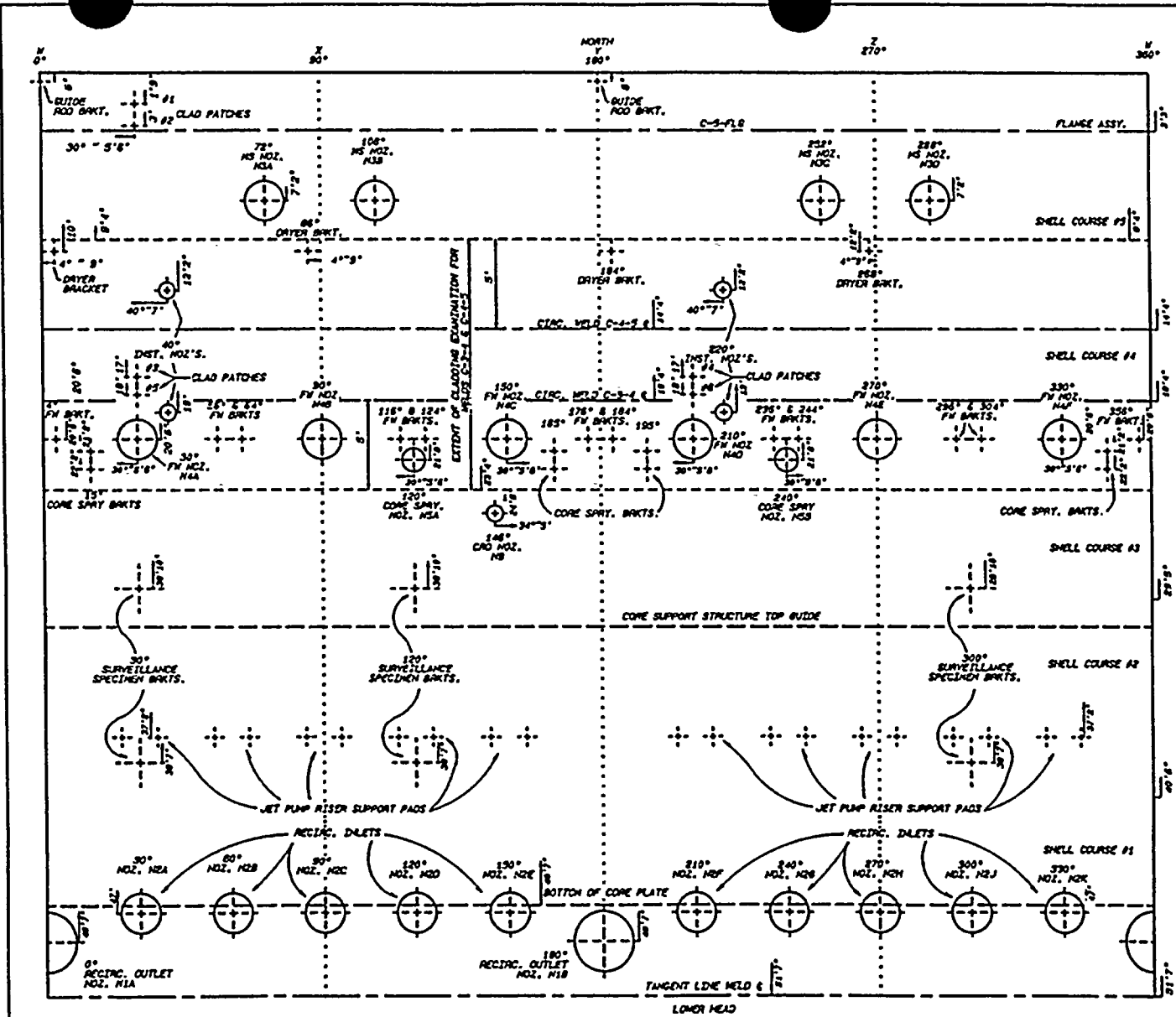
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

ISOMETRICS DRAWINGS SHOWING COMPONENT LOCATION



REFERENCE DRAWING
 500733-001C (N. INSULATION)
 24187-F (BSN)
 122856E

- NOTES:
- FOUR (4) STEAM DRYER HOLDDOWN BRACKETS (NOT SHOWN) ARE LOCATED IN THE VESSEL CLOSURE HEAD 10" UP FROM THE FLANGE. THESE ARE AT AZIMUTH LOCATIONS 41°, 139°, 221°, AND 319°.
 - CODE CATEGORY B-N-1
 RPV INTERIOR
 CODE CATEGORY B-N-2
 RPV-INT ATT BLR
 RPV-INT ATT NBLR
 RPV-CORE SUPPORT
 ○ INTEGRAL ATTACHMENT BELTLINE REGION
 ⊙ INTEGRAL ATTACHMENT NON BELTLINE REGION



PCADAM

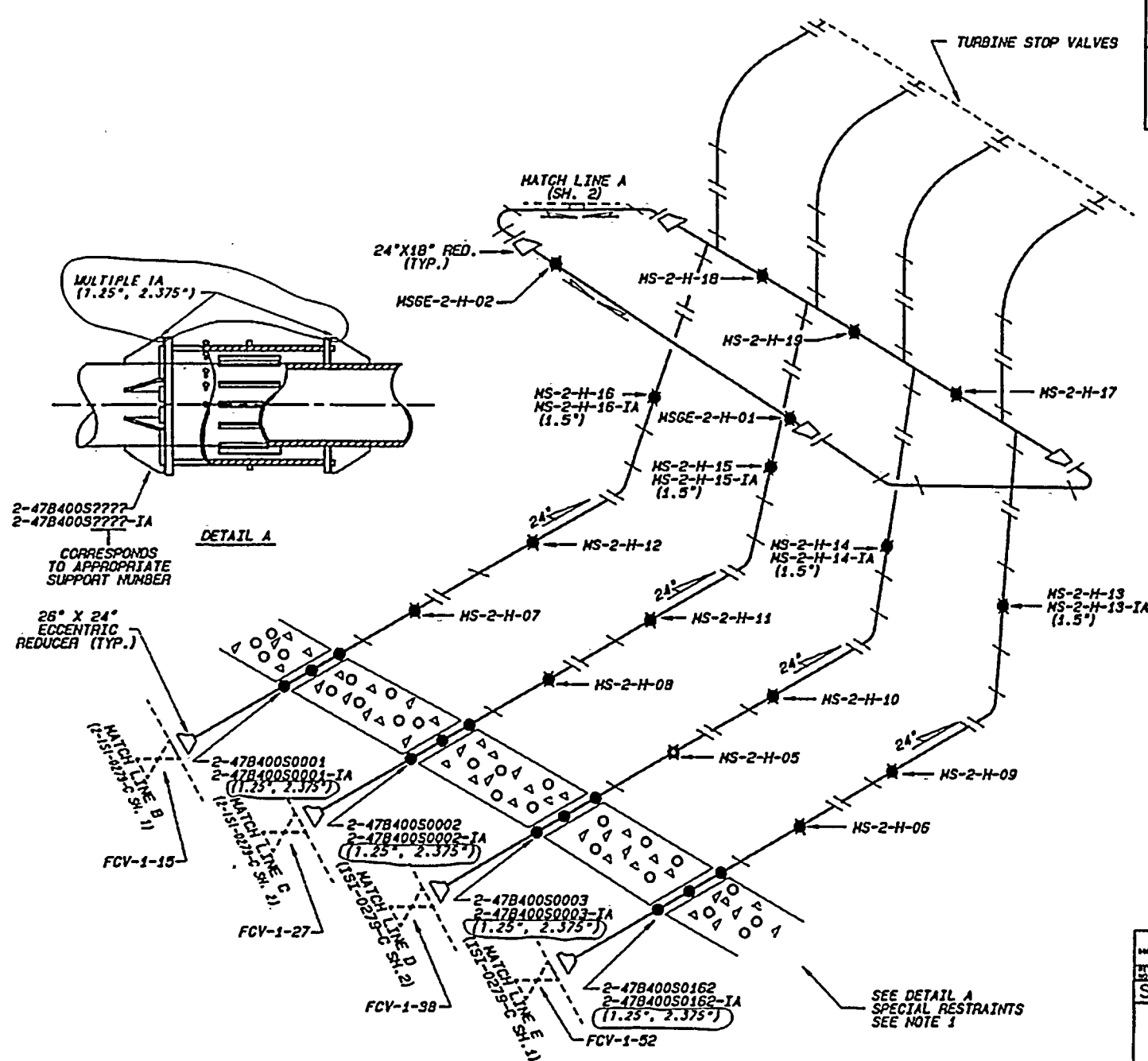
ALL A/D HISTORY RESEARCHED AT R060

DESIGN	REV	DATE	BY	CHKD	APPD	DATE	BY	CHKD	APPD	DATE
S										
TENNESSEE VALLEY AUTHORITY										
BROWNS FERRY NUCLEAR PLANT										
UNIT 2										
REACTOR VESSEL WELD AND NOZZLE LOCATIONS (INTERIOR WALL)										
DATE	BY	DATE	BY	DATE	BY	DATE	BY	DATE	BY	DATE
3-23-80	EDC	3-23-80	GLB	3-23-80	GLB	3-23-80	GLB	3-23-80	GLB	3-23-80
2-CHU-2046-C1000										
CCD										



REFERENCE DRAWINGS
 BP-201-5
 47N400 SH. 1 & 5
 MS6-0021-C (SH. 1) WELD MAP

LEGEND
 ● RIGID HANGER
 * VARIABLE SUPPORT
 ASME CC-2 (EQUIVALENT)



2-47B400S????
 2-47B400S????-IA
 CORRESPONDS
 TO APPROPRIATE
 SUPPORT NUMBER

DETAIL A

26° X 24°
 ECCENTRIC
 REDUCER (TYP.)

MATCH LINE B
 (2-151-0025-C SH. 1)

2-47B400S0001
 2-47B400S0001-IA
 (1.25°, 2.375°)

2-47B400S0002
 2-47B400S0002-IA
 (1.25°, 2.375°)

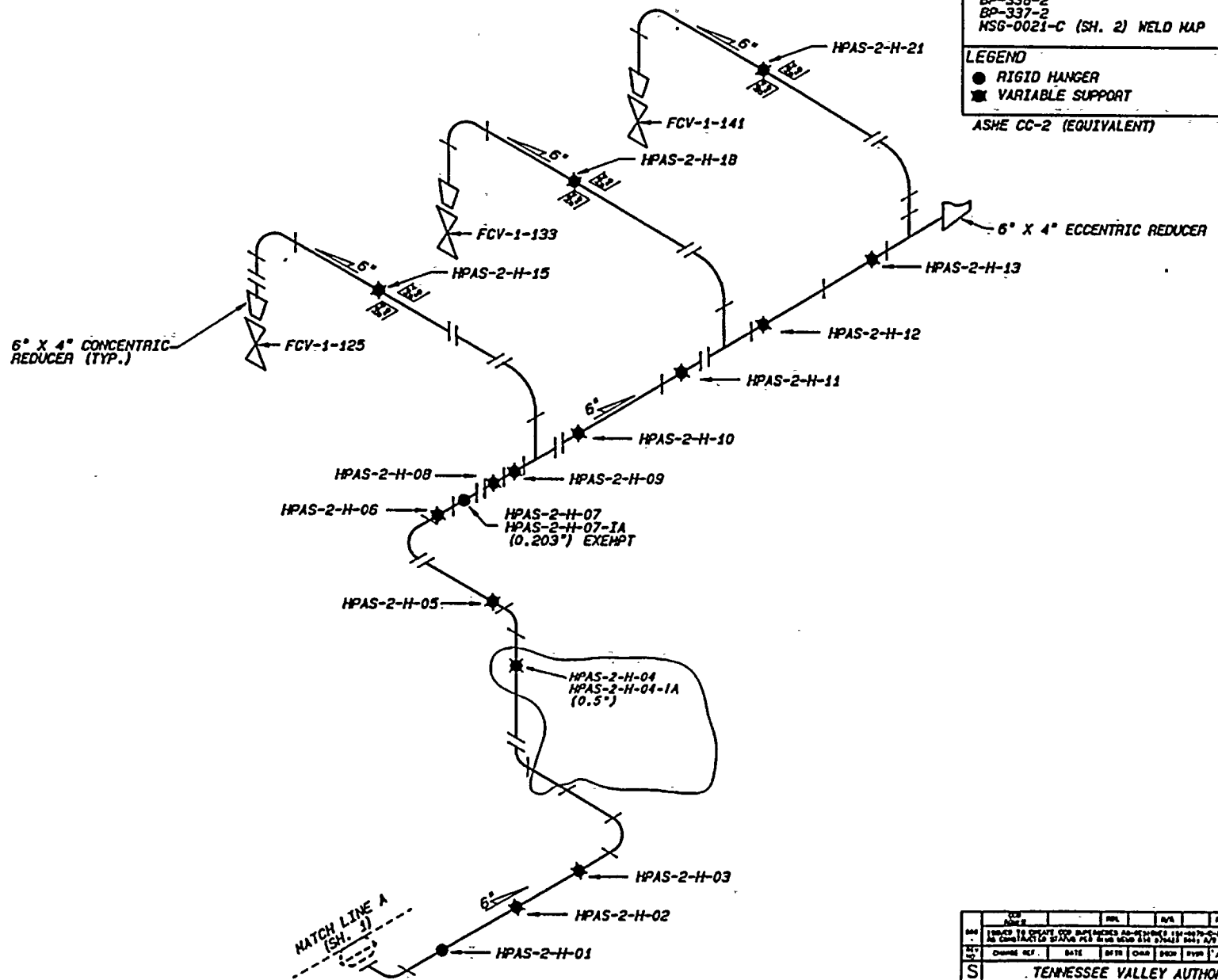
2-47B400S0003
 2-47B400S0003-IA
 (1.25°, 2.375°)

2-47B400S0162
 2-47B400S0162-IA
 (1.25°, 2.375°)

SEE DETAIL A
 SPECIAL RESTRAINTS
 SEE NOTE 1

REV	DATE	BY	CHKD	APPD	REASON
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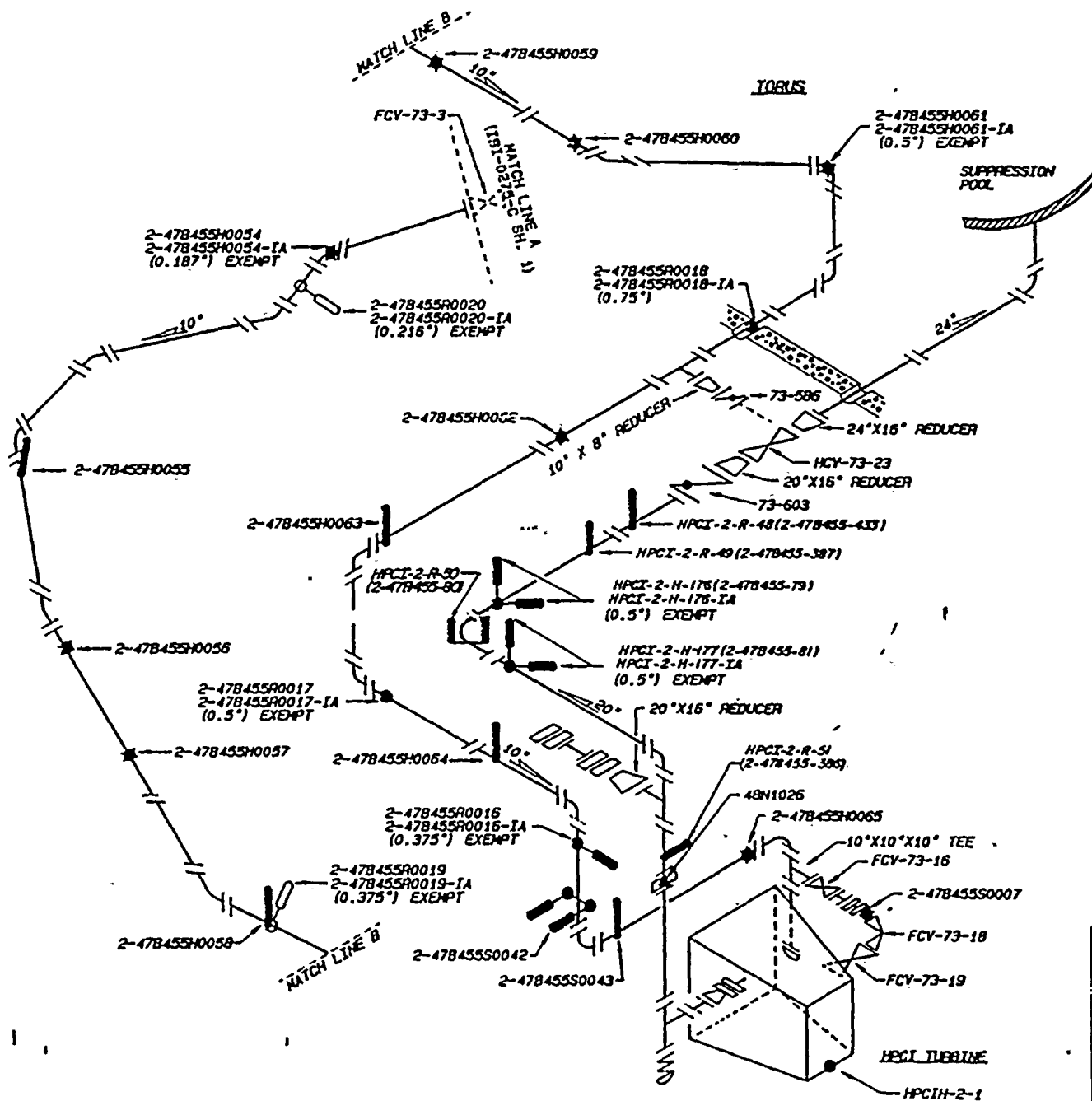
REFERENCE DRAWINGS
 47N400 SH. 1 & 5
 BP-336-2
 BP-337-2
 MSG-0021-C (SH. 2) WELD MAP

LEGEND
 ● RIGID HANGER
 ★ VARIABLE SUPPORT
 ASME CC-2 (EQUIVALENT)

R/F
 RM

REV	DATE	BY	CHKD	APPD	APPD	APPD	APPD
1							
S TENNESSEE VALLEY AUTHORITY							
BROWNS FERRY NUCLEAR PLANT							
UNIT #2							
MAINSTEAM SYSTEM							
SUPPORT LOCATIONS							
CHANGE KEY	SCALE	DATE	BY	CHKD	APPD	APPD	APPD
DATE 12-15-81	SCALE	DATE	BY	CHKD	APPD	APPD	APPD
DATE 02-11-82	SCALE	DATE	BY	CHKD	APPD	APPD	APPD
DATE	SCALE	DATE	BY	CHKD	APPD	APPD	APPD
			GLB				2-151-0079-C(020)
							CCO





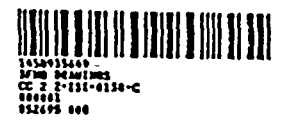
REFERENCE DRAWINGS
 478455-H-6, 7, 203
 ISI-028-C (SH. 1) WELD MAP

LEGEND
 RIGID HANGER
 VARIABLE SUPPORT
 HYDRAULIC SNUBBER
 MECHANICAL SNUBBER
 ANCHOR
 RIGID STRUT

CALCULATION BRANCH
 PROJECT IDENTIFIER:
 CD-02073-86990

ASME CC-2 (EQUIVALENT)

NOTE
 1. SUPPORT DRAWING NUMBERS
 SHOWN IN "I" DRAWING
 MAY HAVE MULTIPLE SHEET
 NUMBERS.



CD/IT	ISS	CHK	APP	REV	DATE	BY	CHK	DATE
0	ISSUED TO CREATE CD, SUPERSEDES APPROJ-0130-C-1 AND,							
0	REVISED SUPPORTS AND SUPPORT NUMBERS, ADDED							
0	REFERENCE CALCULATION, ISI REVISION							
TENNESSEE VALLEY AUTHORITY								
BROWNS FERRY NUCLEAR PLANT								
UNIT # 2								
HIGH PRESSURE COOLANT INJECTION SYSTEM								
SUPPORT LOCATIONS								
DATE	BY	DATE	DATE	DATE	DATE	DATE	DATE	DATE
GLB						2-151-0130-C 000		
CCD								

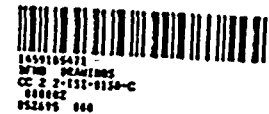
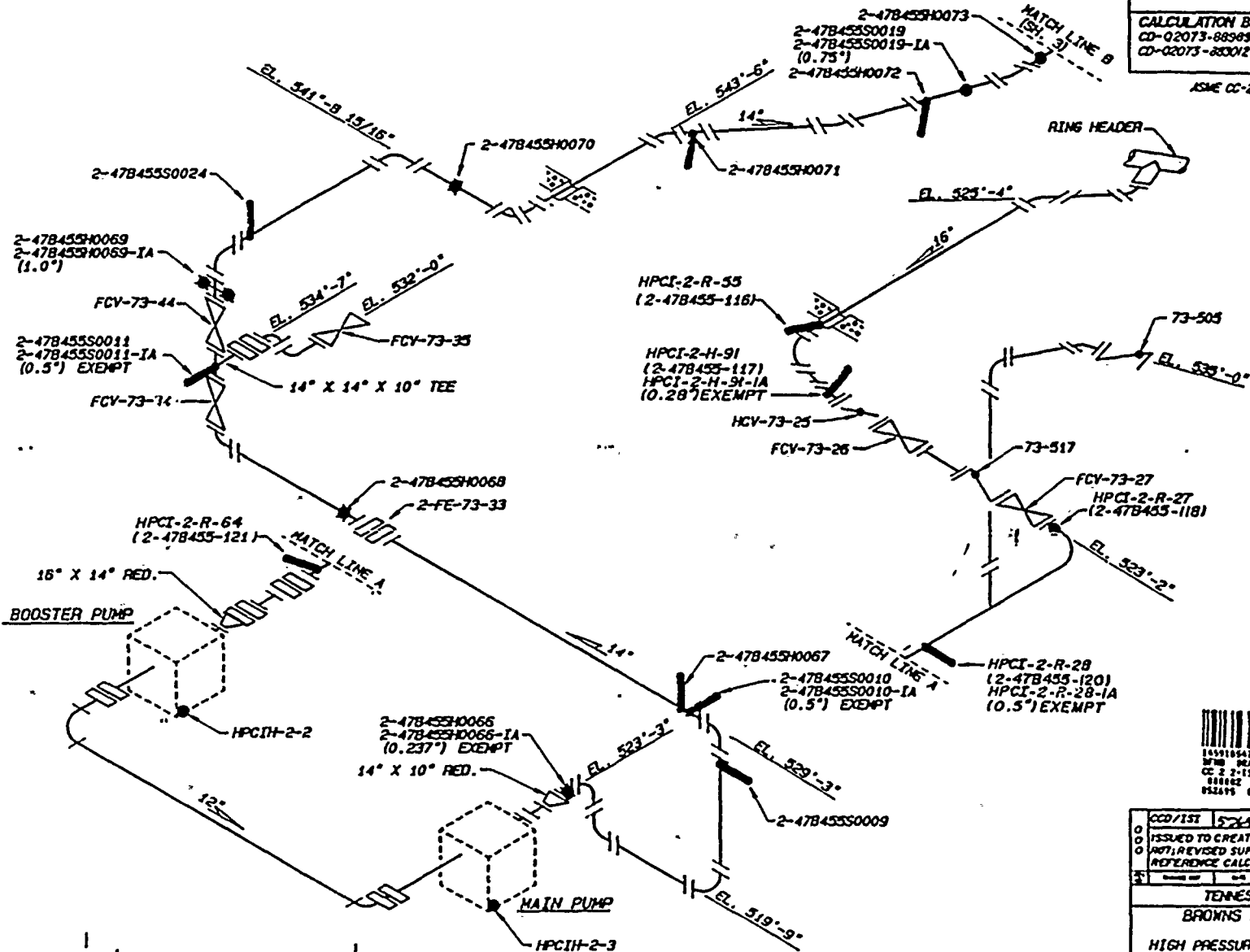


REFERENCE DRAWINGS
47B455-H SERIES
ISI-0128-C (SH. 2) WELLS

LEGEND
● RIGID HANGER
■ VARIABLE SUPPORT
— RIGID STRUT

CALCULATION BRANCH / PROJECT IDENTIFIERS:
CD-02073-88509
CD-02073-88002

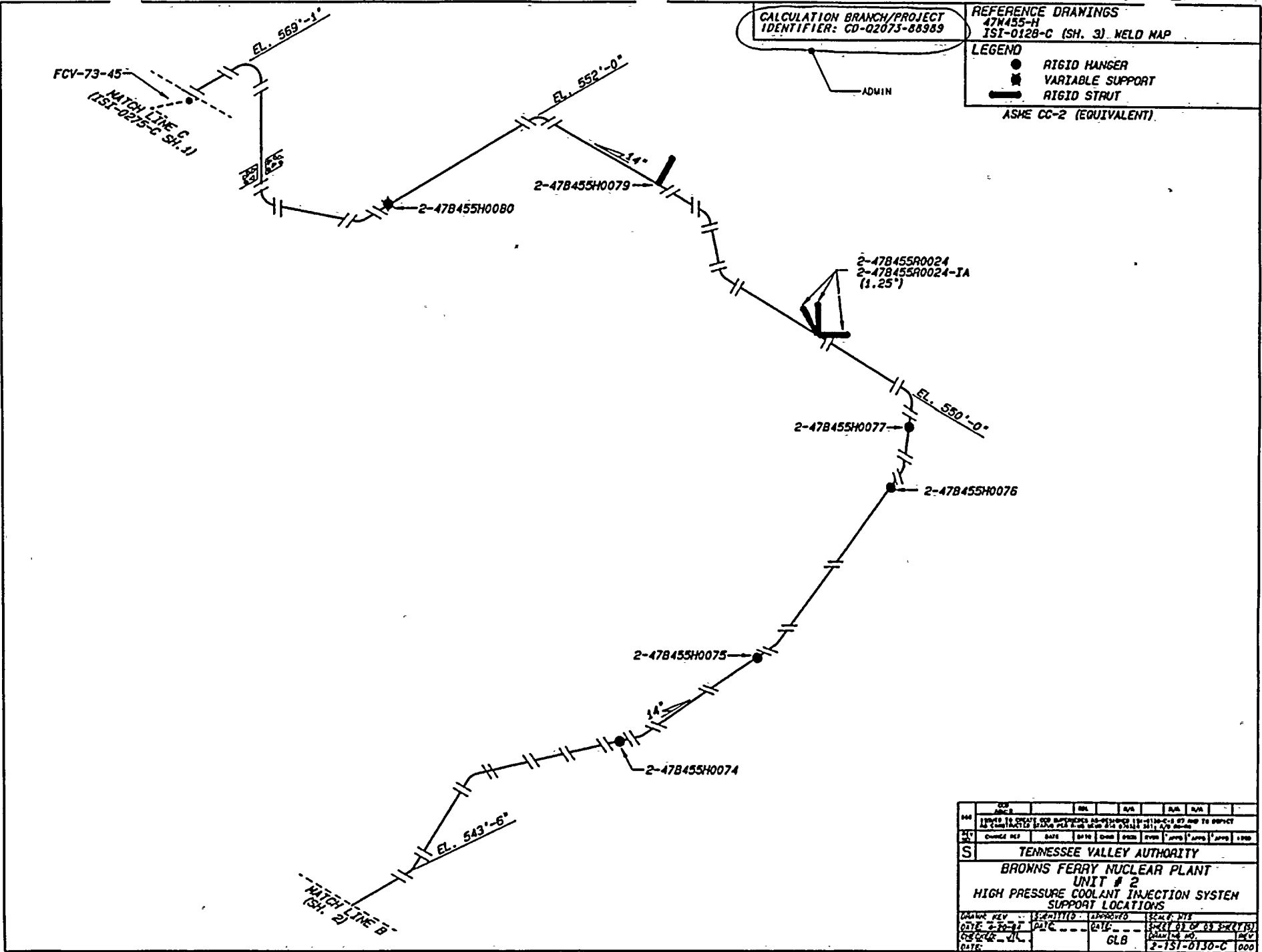
ASME CC-2 (EQUIVALENT)



ISSUED TO CREATE CCD, SUPERSEDES A0151-0130-C-2	DATE: 2-20-93	SCALE: 1:1
NOT REVISION SUPPORT IDENTIFIERS AND ADDED REFERENCE CALCULATION, 1151 REVISION)	DATE: 2-20-93	SCALE: 1:1
TENNESSEE VALLEY AUTHORITY		
BROWNS FERRY NUCLEAR PLANT		
UNIT # 2		
HIGH PRESSURE COOLANT INJECTION SYSTEM		
SUPPORT LOCATIONS		
DESIGN: JBY	REVISION: 001	SCALE: 1:1
DATE: 2-20-93	DATE: 2-20-93	SCALE: 1:1
DESIGN: JBY	DATE: 2-20-93	SCALE: 1:1
DATE: 2-20-93	DATE: 2-20-93	SCALE: 1:1
GLB		
2-151-0130-C		
CCD		

Rev. 000





CALCULATION BRANCH/PROJECT
IDENTIFIER: CD-Q2075-88989

REFERENCE DRAWINGS
47N455-H
ISI-0128-C (SH. 3) WELD MAP

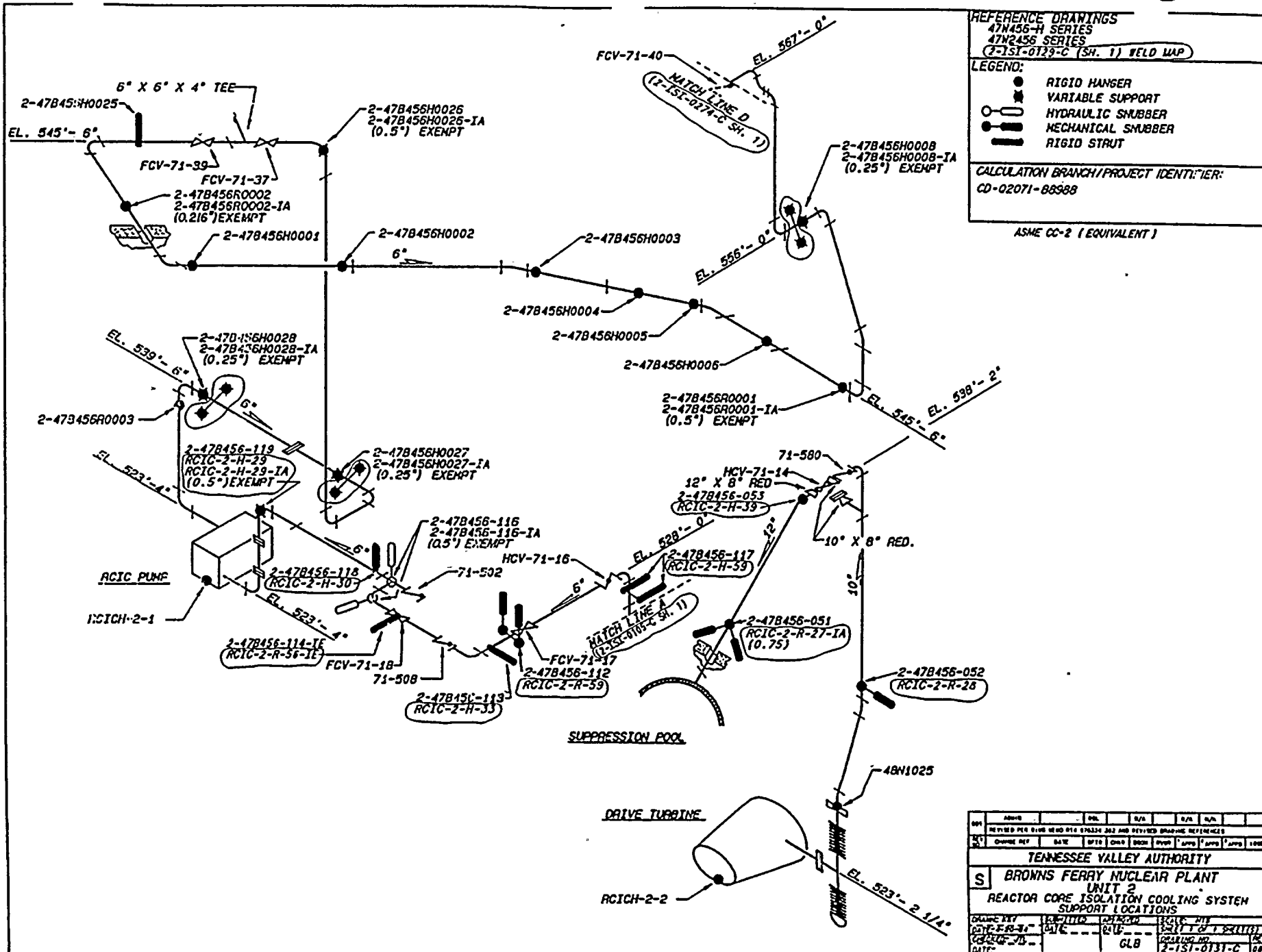
LEGEND
 ● RIGID HANGER
 ■ VARIABLE SUPPORT
 — RIGID STRUT
 ASME CC-2 (EQUIVALENT)

REV	DATE	BY	CHKD	APPD	DESCRIPTION
000					ISSUED TO CREATE FOR SUPPORTS AS DESIGNED BY 1110-C-107 AND TO SERVICE ALL COMPONENTS WITHIN THE AREA OF THE SYSTEM
001					CHANGED PER 1110-C-107
002					CHANGED PER 1110-C-107
003					CHANGED PER 1110-C-107
004					CHANGED PER 1110-C-107
005					CHANGED PER 1110-C-107
006					CHANGED PER 1110-C-107
007					CHANGED PER 1110-C-107
008					CHANGED PER 1110-C-107
009					CHANGED PER 1110-C-107
010					CHANGED PER 1110-C-107
011					CHANGED PER 1110-C-107
012					CHANGED PER 1110-C-107
013					CHANGED PER 1110-C-107
014					CHANGED PER 1110-C-107
015					CHANGED PER 1110-C-107
016					CHANGED PER 1110-C-107
017					CHANGED PER 1110-C-107
018					CHANGED PER 1110-C-107
019					CHANGED PER 1110-C-107
020					CHANGED PER 1110-C-107

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR PLANT
 UNIT # 2
 HIGH PRESSURE COOLANT INJECTION SYSTEM
 SUPPORT LOCATIONS

DATE	REV	BY	CHKD	APPD	DESCRIPTION
01/11/10	001	GLB			ISSUED FOR CONSTRUCTION
01/11/10	002	GLB			ISSUED FOR CONSTRUCTION
01/11/10	003	GLB			ISSUED FOR CONSTRUCTION
01/11/10	004	GLB			ISSUED FOR CONSTRUCTION
01/11/10	005	GLB			ISSUED FOR CONSTRUCTION
01/11/10	006	GLB			ISSUED FOR CONSTRUCTION
01/11/10	007	GLB			ISSUED FOR CONSTRUCTION
01/11/10	008	GLB			ISSUED FOR CONSTRUCTION
01/11/10	009	GLB			ISSUED FOR CONSTRUCTION
01/11/10	010	GLB			ISSUED FOR CONSTRUCTION
01/11/10	011	GLB			ISSUED FOR CONSTRUCTION
01/11/10	012	GLB			ISSUED FOR CONSTRUCTION
01/11/10	013	GLB			ISSUED FOR CONSTRUCTION
01/11/10	014	GLB			ISSUED FOR CONSTRUCTION
01/11/10	015	GLB			ISSUED FOR CONSTRUCTION
01/11/10	016	GLB			ISSUED FOR CONSTRUCTION
01/11/10	017	GLB			ISSUED FOR CONSTRUCTION
01/11/10	018	GLB			ISSUED FOR CONSTRUCTION
01/11/10	019	GLB			ISSUED FOR CONSTRUCTION
01/11/10	020	GLB			ISSUED FOR CONSTRUCTION





REFERENCE DRAWINGS
 47B456-H SERIES
 47B456 SERIES
 (2-151-0129-C (SH. 1) WELD MAP)

- LEGEND:
- RIGID HANGER
 - VARIABLE SUPPORT
 - HYDRAULIC SNUBBER
 - MECHANICAL SNUBBER
 - RIGID STRUT

CALCULATION BRANCH / PROJECT IDENTIFIER:
 CD-02071-88988

ASME CC-2 (EQUIVALENT)

REV	DATE	BY	CHKD	APP'D	DESC
001					REVISED PER STATE HEAD 916 6/22/84 AND REVISED DRAWING REQUIREMENTS
002					CHANGED REV
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
REACTOR CORE ISOLATION COOLING SYSTEM					
SUPPORT LOCATIONS					
DRAWN BY	DATE	CHKD BY	DATE	APP'D BY	DATE
GLB	6/16	GLB	6/16	GLB	6/16
DATE		DATE		DATE	
					CCD



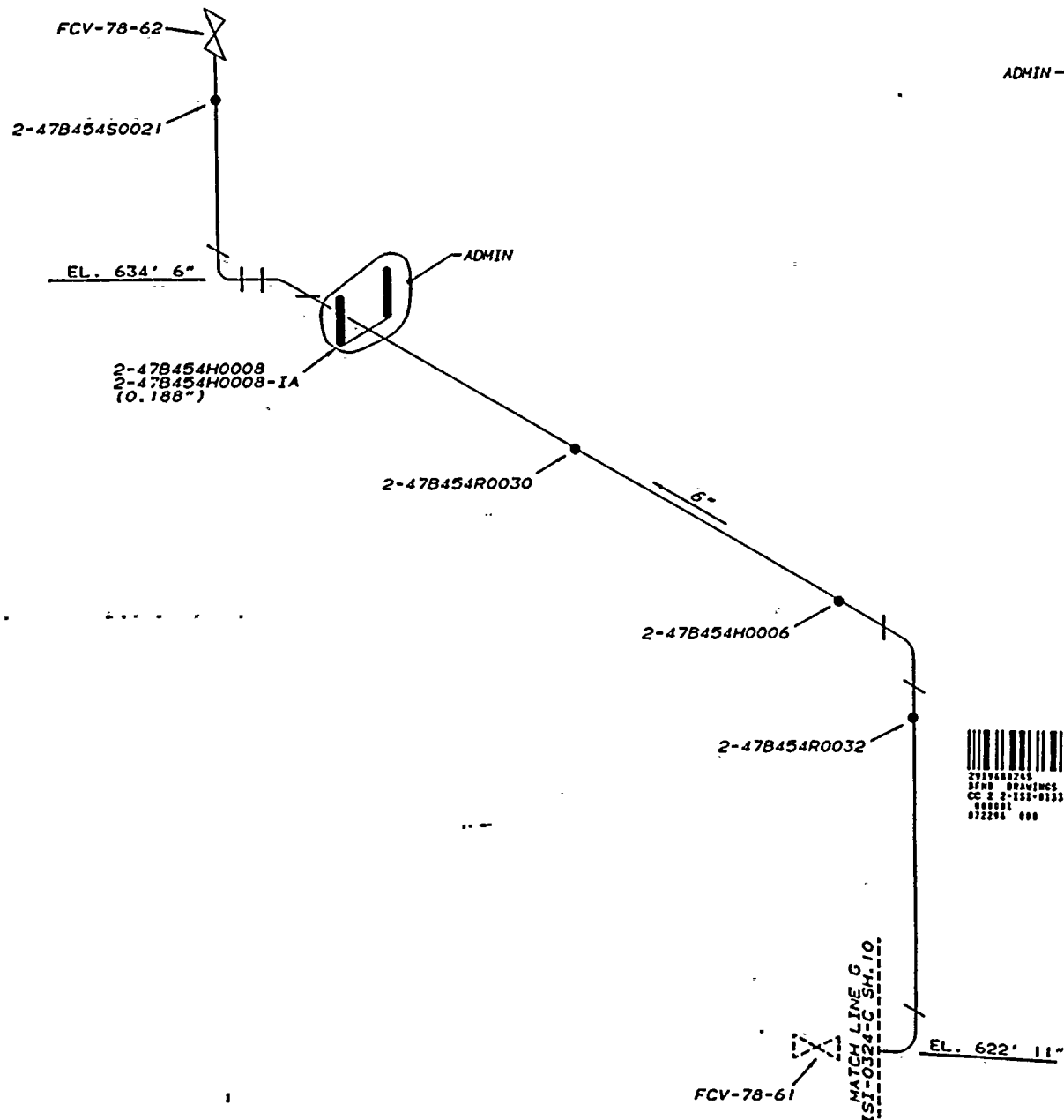
REFERENCE DRAWING
0-47W454-1, -3

LEGEND

- RIGID SUPPORT
- RIGID STRUT

CALCULATION BRANCH/PROJECT IDENTIFIER:
CD-02078-881002

ASME CC-3 (EQUIVALENT)



231760245
3FND DRAWINGS
CC 2 2-151-0133-C
188881
872276 888

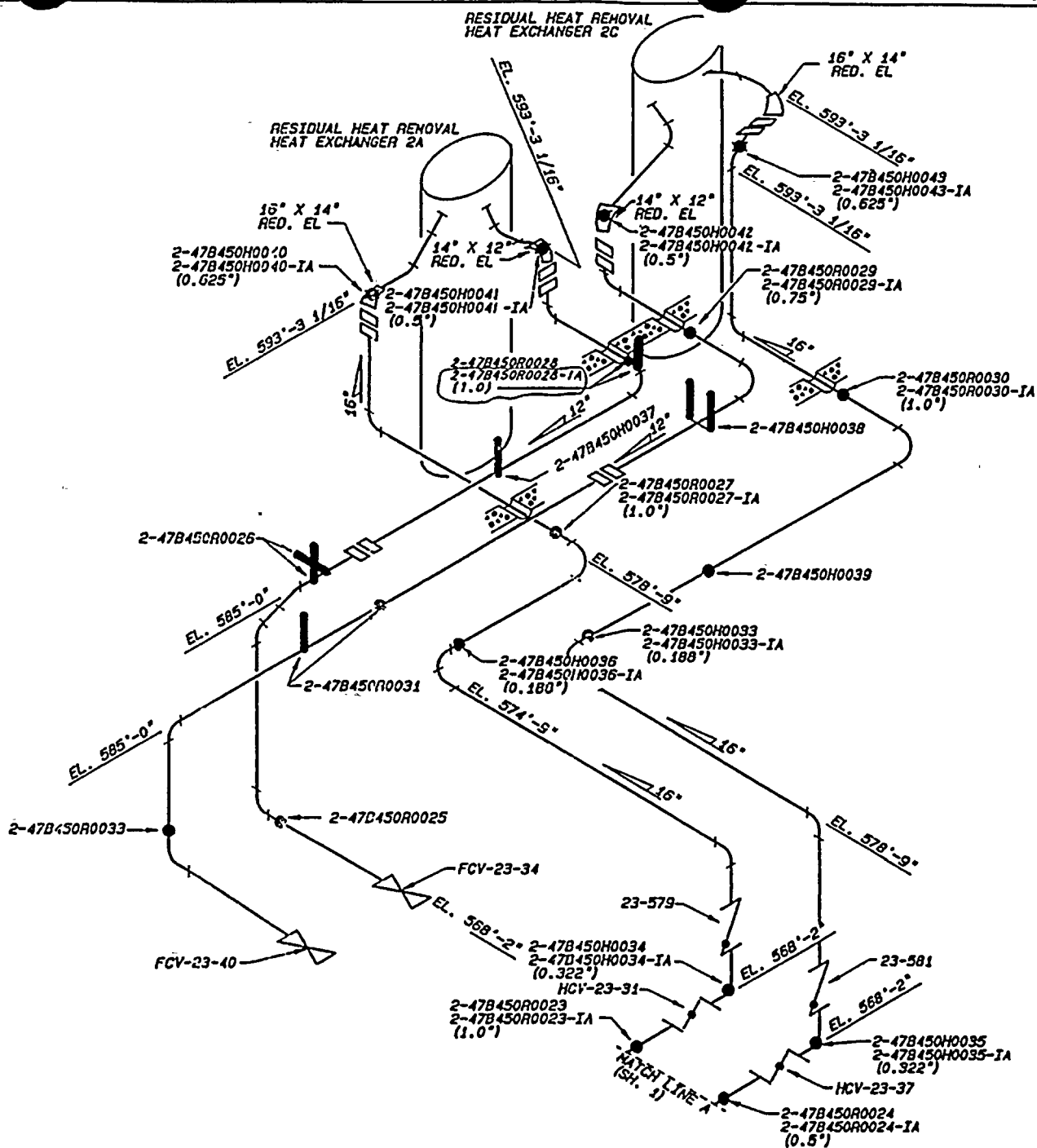
CDW/ADP/	722-96	W/JK	RD	REV	REV	REV	REV	REV	REV
ISSUED TO CREATE CDW	REWORKED AS SHOWN IN THE CDW								
ALL CONSTRUCTION	SHOULD BE DONE IN ACCORDANCE WITH THE CDW								
CONSTRUCTION	SHOULD BE DONE IN ACCORDANCE WITH THE CDW								
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT UNIT 2									
FUEL POOL COOLING SYSTEM SUPPORT LOCATIONS									
DRAWN: KEY	DATE: 12-13-83	SCALE: N/A	CDW/ADP/	REV	REV	REV	REV	REV	REV
CHECKED: JTL	APPROVED: QLB	CDW/ADP/	REV	REV	REV	REV	REV	REV	REV
SUBMITTED:			2 ISI-0133-C			000			
CCD									

1551 FILE#0007216 23
 FILE#0007216 23
 ACCESS#0007216 23
 1551 FILE#0007216 23
 FILE#0007216 23
 ACCESS#0007216 23

CADAM

ALL A/D HISTORY RESEARCHED AT ROOM





REFERENCE DRAWINGS

2-47E450-2
0-47N450-4

LEGEND

- RIGID HANGER
- ✱ VARIABLE SUPPORT
- RIGID STRUT

CALCULATION BRANCH / PROJECT IDENTIFIER

CD-02023-891351
CD-02023-891352
CD-02023-891353

ASME CC-3 (EQUIVALENT)

APPD	REV	N/A	N/A	N/A
DATE	BY	CHKD	APPD	DATE
TENNESSEE VALLEY AUTHORITY				
BROWNS FERRY NUCLEAR PLANT				
UNIT # 2				
RESIDUAL HEAT REMOVAL SERVICE WATER SYSTEM				
SUPPORT LOCATIONS				
DATE	BY	DATE	BY	DATE
02/17/83	JTC	02/17/83	GLB	02/17/83
03/04/83	JTC	03/04/83	GLB	03/04/83
03/04/83	JTC	03/04/83	GLB	03/04/83
CCD				

M/F
R-202



CALCULATION BRANCH / PROJECT IDENTIFIERS:

CD-02023-880976
CD-02023-891348

REFERENCE DRAWINGS

2-47E450-2
0-47N450-4

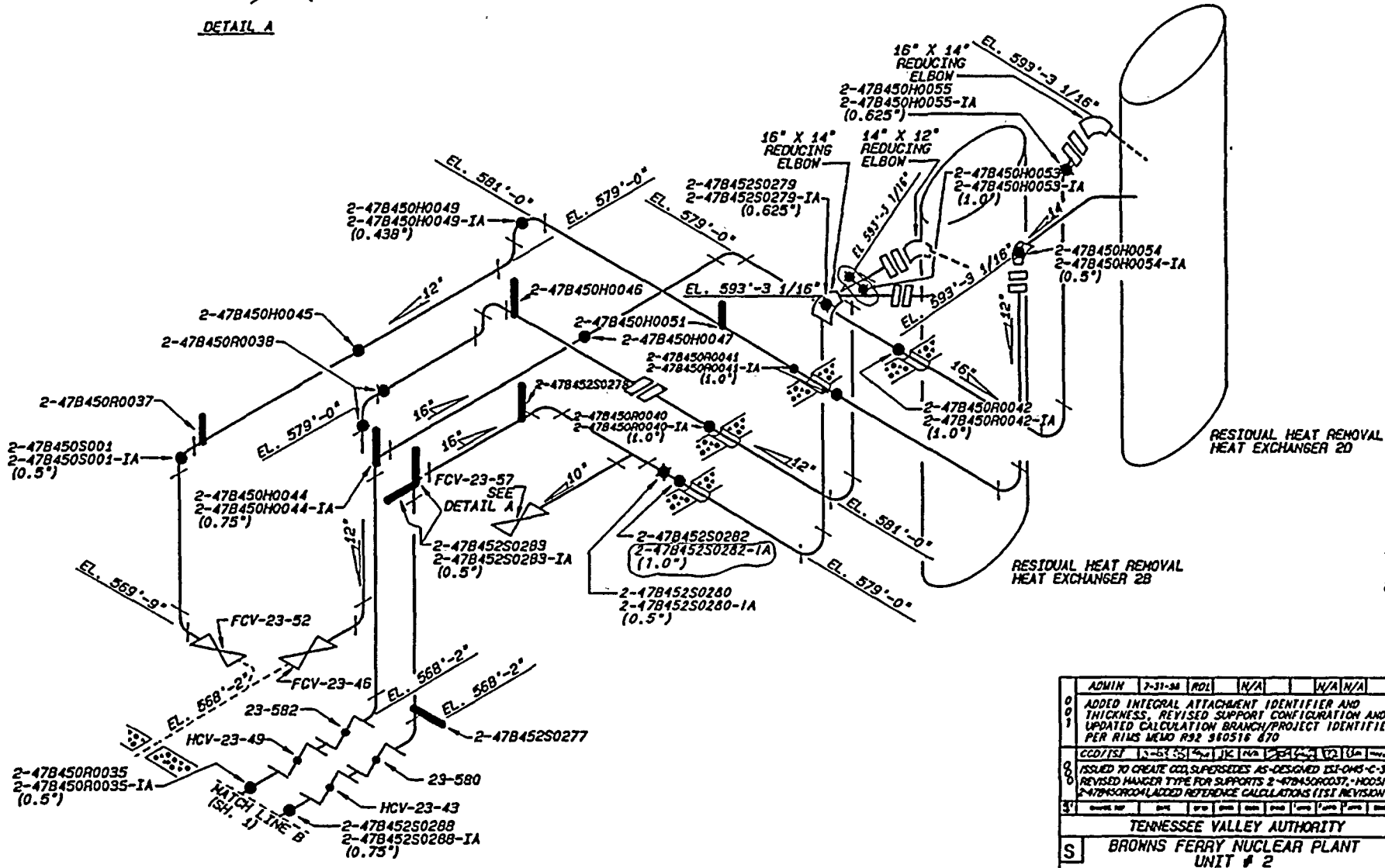
LEGEND

- RIGID HANGER
- VARIABLE SUPPORT
- RIGID STRUT

ASME CC-3 (EQUIVALENT)



DETAIL A



ADMIN	7-31-88	ROL	N/A	N/A	N/A	N/A
0	ADDED INTEGRAL ATTACHMENT IDENTIFIER AND THICKNESS, REVISED SUPPORT CONFIGURATION AND UPDATED CALCULATION BRANCH/PROJECT IDENTIFIER PER RIHS MEMO R92 860516 870					
1	ISSUED TO CREATE CCD, SUPERSEDES AS-DESIGNED EST-045-C-3 ROS, REVISED HANGER TYPE FOR SUPPORTS 2-47B450R0037, -H0051 AND 2-47B450R0042 (SEE REFERENCE CALCULATIONS (TSI REVISION))					
31	DATE	BY	CHK	APP	DES	REV
TENNESSEE VALLEY AUTHORITY						
S BROWNS FERRY NUCLEAR PLANT UNIT # 2 RESIDUAL HEAT REMOVAL SERVICE WATER SYSTEM SUPPORT LOCATIONS						
DATE	REV	ISSUED	APPROVED	SCALE	NIS	
DATE	3-18-84	DATE		SHEET	3 OF 3	DATE
DATE		DATE	GLB	DATE		DATE
DATE		DATE		DATE		DATE
2-TS1-045-C						1001
CCD						



NOTE:

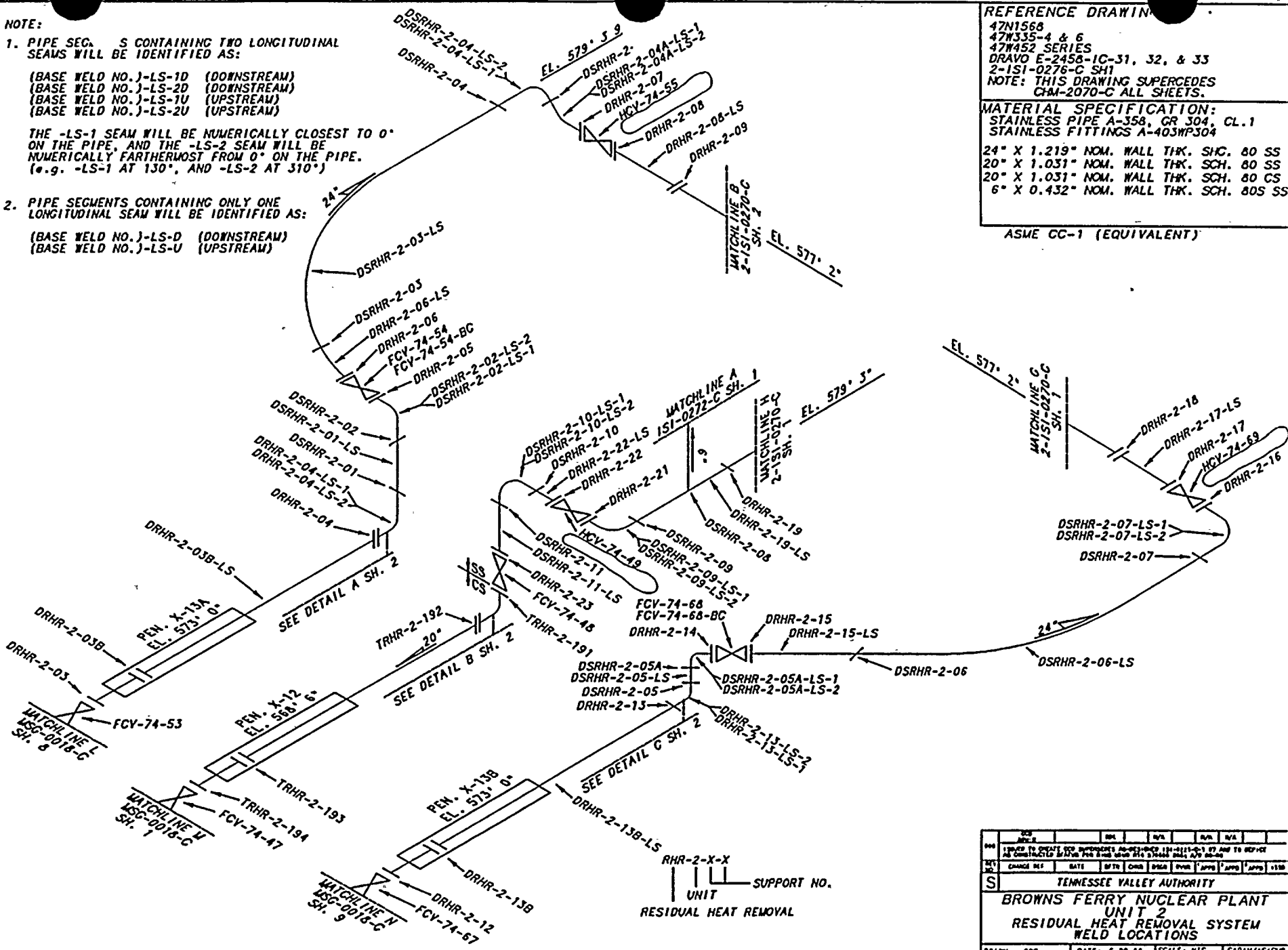
1. PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:

(BASE WELD NO.)-LS-1D (DOWNSTREAM)
 (BASE WELD NO.)-LS-2D (DOWNSTREAM)
 (BASE WELD NO.)-LS-1U (UPSTREAM)
 (BASE WELD NO.)-LS-2U (UPSTREAM)

THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE, AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHERMOST FROM 0° ON THE PIPE.
 (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)

2. PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS:

(BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)



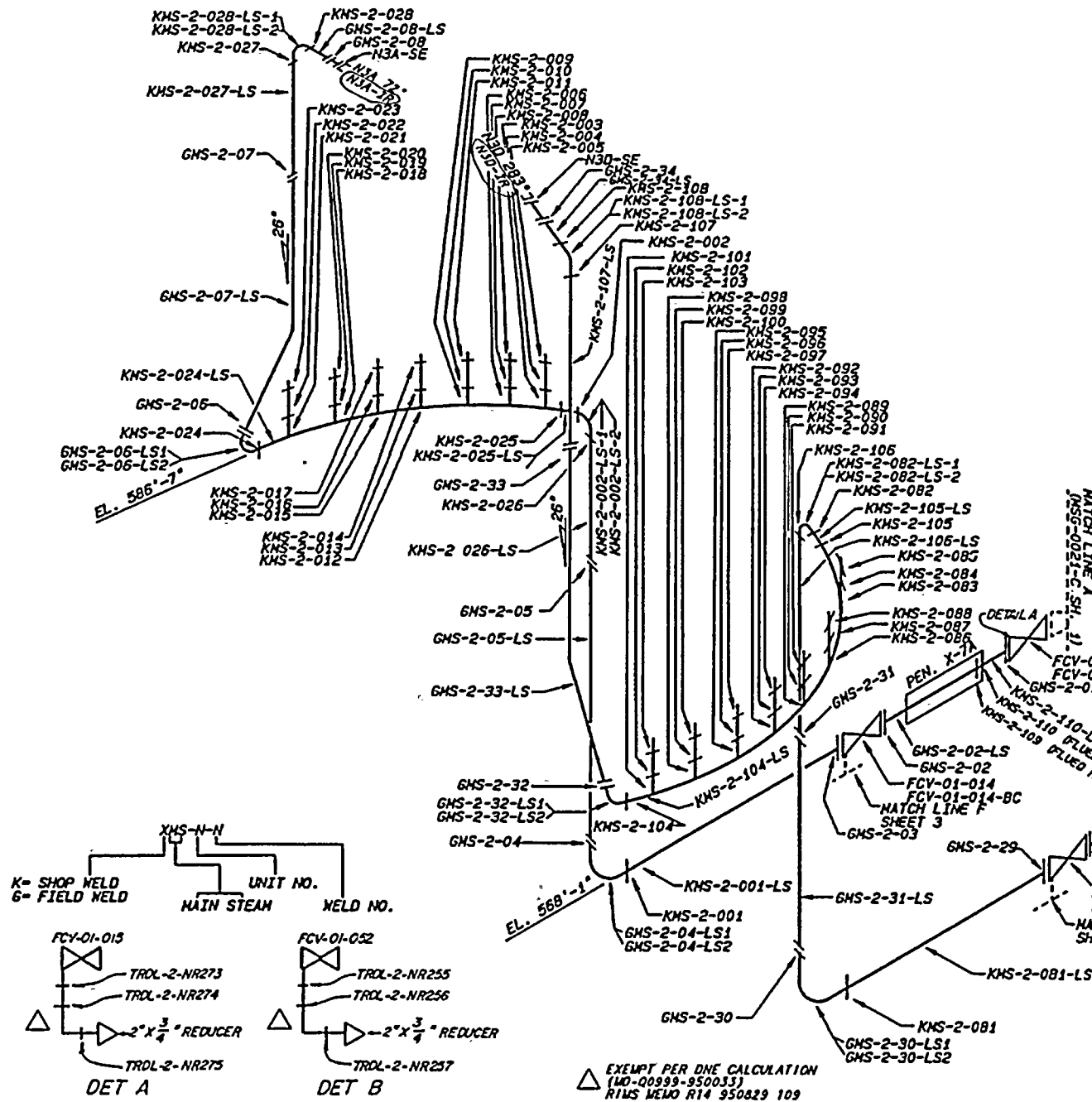
REFERENCE DRAWING:
 47N1568
 47W335-4 & 6
 47W452 SERIES
 DRAVO E-2458-1C-31, 32, & 33
 2-1S1-0276-C SH1
 NOTE: THIS DRAWING SUPERCEDES CHM-2070-C ALL SHEETS.

MATERIAL SPECIFICATION:
 STAINLESS PIPE A-358, GR 304, CL.1
 STAINLESS FITTINGS A-403MP304
 24" X 1.219" NOM. WALL THK. SCH. 80 SS
 20" X 1.031" NOM. WALL THK. SCH. 80 SS
 20" X 1.031" NOM. WALL THK. SCH. 80 CS
 6" X 0.432" NOM. WALL THK. SCH. 80S SS

ASME CC-1 (EQUIVALENT)

REV	DATE	BY	CHKD	APPD	Q/A
000					
001					
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099					
100					





REFERENCE DRAWINGS:
 47K1757
 47N335-1
 KELLOGG 729E229
 CHN-2087-C (SH. 1) SUPPORT MAP
 NOTE: THIS DRAWING SUPERCEDES
 CHN-2069-C (SH. 1)

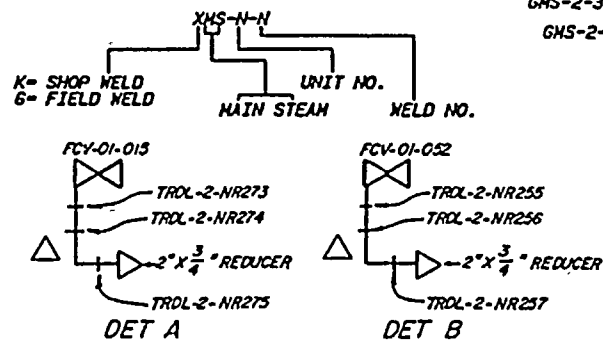
PIPE DATA
 ASME CC-1 (EQUIVALENT)
 ASTM A-155 KC 70
 26" X 0.950 NOM. WALL THK. (CS)
 6" X 0.719 NOM. WALL THK. (SCH. 160 CS)

NOTES:
 ALL FIELD WELDS WERE MADE BY TVA

NOTE:
 1. PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD NO.)-LS-1D (DOWNSTREAM)
 (BASE WELD NO.)-LS-2D (DOWNSTREAM)
 (BASE WELD NO.)-LS-1U (UPSTREAM)
 (BASE WELD NO.)-LS-2U (UPSTREAM)

THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHEST FROM 0° ON THE PIPE. (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)

2. PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS:
 (BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)



△ EXEMPT PER ONE CALCULATION
 (MO-00999-950033)
 RIMS MEMO R14 950829 109

NO.	REVISED PER	DATE	BY	CHKD	APPD	DATE	BY	CHKD	APPD
1	CHANGED REF								
TENNESSEE VALLEY AUTHORITY									
S BRONNS FERRY NUCLEAR PLANT UNIT 2 MAIN STEAM SYSTEM WELD LOCATIONS									
DESIGNED BY	EDC	DATE	8-10-88	CHECKED BY	GLB	DATE	8-10-88	APPROVED BY	2-IST-022-C
CCD									

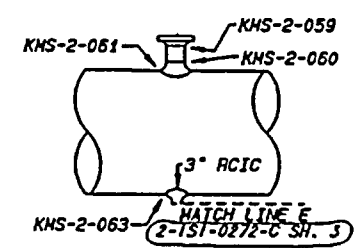
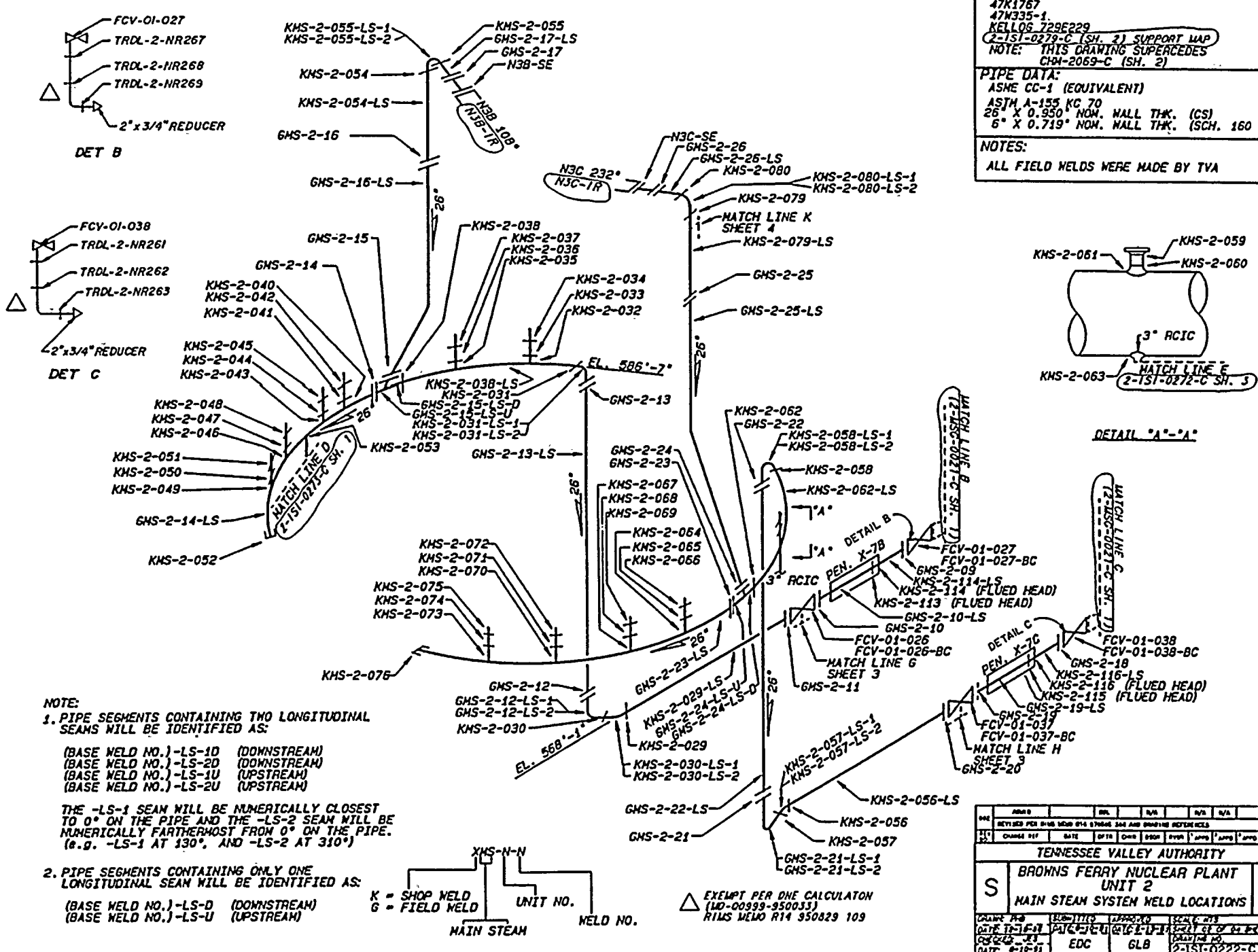
MS
 R4



REFERENCE DRAWINGS:
 47K1767
 47N335-1
 KELLOGG 729E229
 2-151-0279-C (SH. 2) SUPPORT MAP
 NOTE: THIS DRAWING SUPERCEDES
 CHM-2069-C (SH. 2)

PIPE DATA:
 ASME CC-1 (EQUIVALENT)
 ASTM A-155 KC 70
 26" X 0.950" NOM. WALL THK. (CS)
 6" X 0.719" NOM. WALL THK. (SCH. 160 CS)

NOTES:
 ALL FIELD WELDS WERE MADE BY TVA



DET B

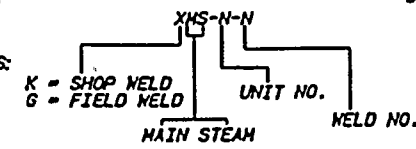
DET C

DETAIL "A"- "A"

NOTE:
 1. PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD NO.)-LS-1D (DOWNSTREAM)
 (BASE WELD NO.)-LS-2D (DOWNSTREAM)
 (BASE WELD NO.)-LS-1U (UPSTREAM)
 (BASE WELD NO.)-LS-2U (UPSTREAM)

THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHERMOST FROM 0° ON THE PIPE. (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)

2. PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS:
 (BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)



EXEMPT PER DNE CALCULATOR
 (MD-00993-950033)
 RIUS MEMO R14 350829 109

NO.	REV.	DATE	BY	CHKD.	APP'D.	REASON
1						
2						
3						
4						
5						
6						
7						
8						
9						
10						

TENNESSEE VALLEY AUTHORITY

S BROWNS FERRY NUCLEAR PLANT
 UNIT 2
 MAIN STEAM SYSTEM WELD LOCATIONS

DATE: 1-14-81	BY: EDC	CHKD: 6LB	APP'D: [Signature]
DATE: 6-10-81	BY: EDC	CHKD: 6LB	APP'D: [Signature]



REFERENCE DRAWING
B&W 122863

NOTE: THIS DRAWING SUPERSEDES
CHM-2001-C (UNIT 2)
ASME CC-1 (EQUIVALENT)

0° - SOUTH

300°

60°

270° - EAST Z

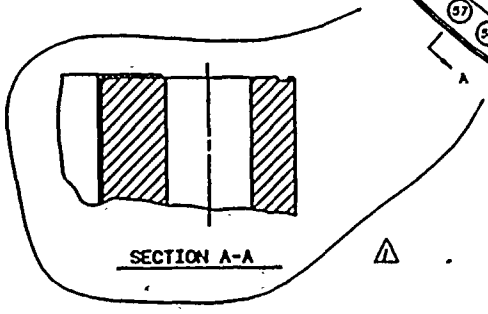
92 6.75" TAPPED HOLES ON
267.25" DIA. B.C. REF.

X 90° - WEST

240°

120°

NUMBERS (01 THRU 92) ARE PREFIXED BY:
LIGAMENTS (THREADS)...RPV-LIGS-2-
STUDS.....RPV-STUDS-2-
NUTS.....RPV-NUTS-2-
WASHERS.....RPV-WASH-2-
BUSHINGS.....RPV-BUSH-2-



180° - NORTH

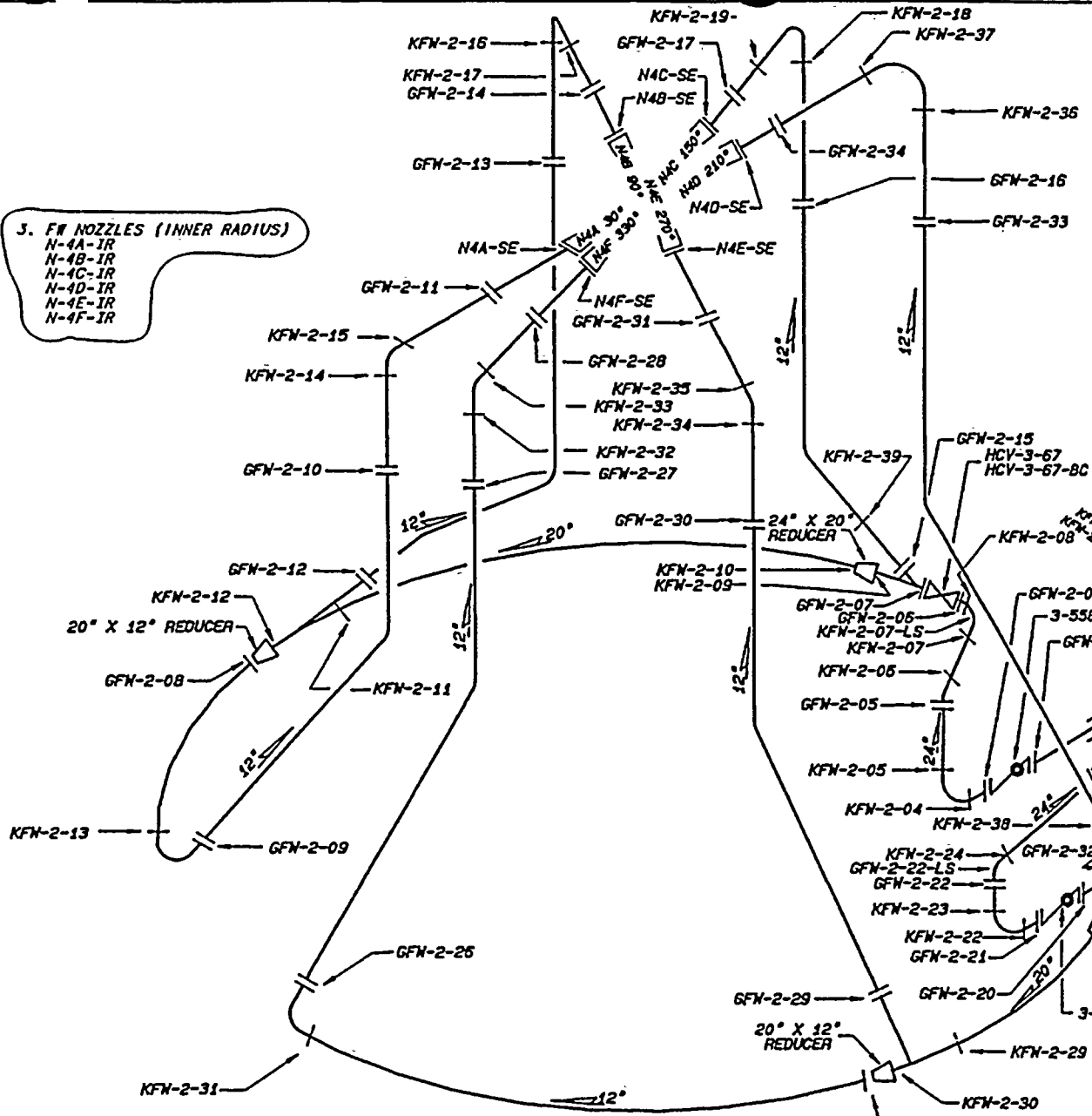
Sheet 01 of 01
ISI-0266-C Rev. 01

1	PHB	2/23	1/28	1/28	3/13/82
ADD BUSHING PREFIX, CHANGE NOTE, REF. DWG., ADD FLANGE PROFILE					
REV.	BY	CHECKED	DESIGNED	APPROVED	DATE
HARDWARE	10M 5085	SOFTWARE	CAOM	USER	IS10P
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 REACTOR VESSEL STUD LOCATIONS					
DRAWN	PHB	DESIGNED	APPROVED	SCALE	NONE
DATE	8-20-82	DATE	8-3-82	DATE	8-17-82
CHECKED	JES	ETC	DB	DATE	8-2-82
ISI-0266-C-01					

ISI-0266-C



J. FW NOZZLES (INNER RADIUS)
 N-4A-IR
 N-4B-IR
 N-4C-IR
 N-4D-IR
 N-4E-IR
 N-4F-IR

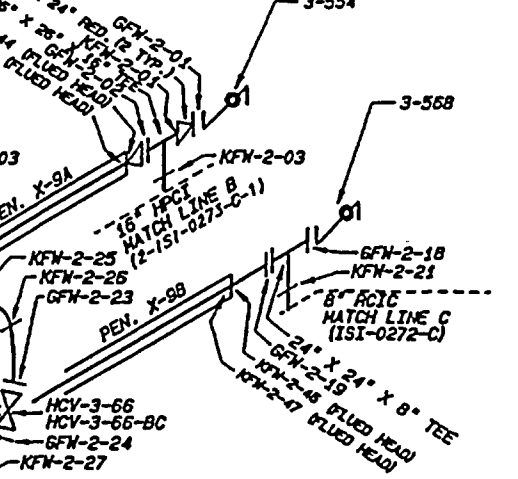


REFERENCE DRAWINGS
 47K1545-1
 47W335-2
 GE 729E152
 ISI-0277-C (SH. 1) SUPPORT MAP
 THIS DRAWING SUPERCEDES CHN-2067-C
 ALL SHEETS.

MATERIAL SPECIFICATIONS
 ASTM A-106 OR B SCH. 100 OR ASTM A-135 OR K07700
 8\"/>

ASME CC-1 (EQUIVALENT)

- NOTES**
1. GE FIELD WELDS WERE MADE IN THE FIELD BY TVA.
 2. PIPE SEGMENT LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD)-LS-D (DOWNSTREAM)
 (BASE WELD)-LS-U (UPSTREAM)

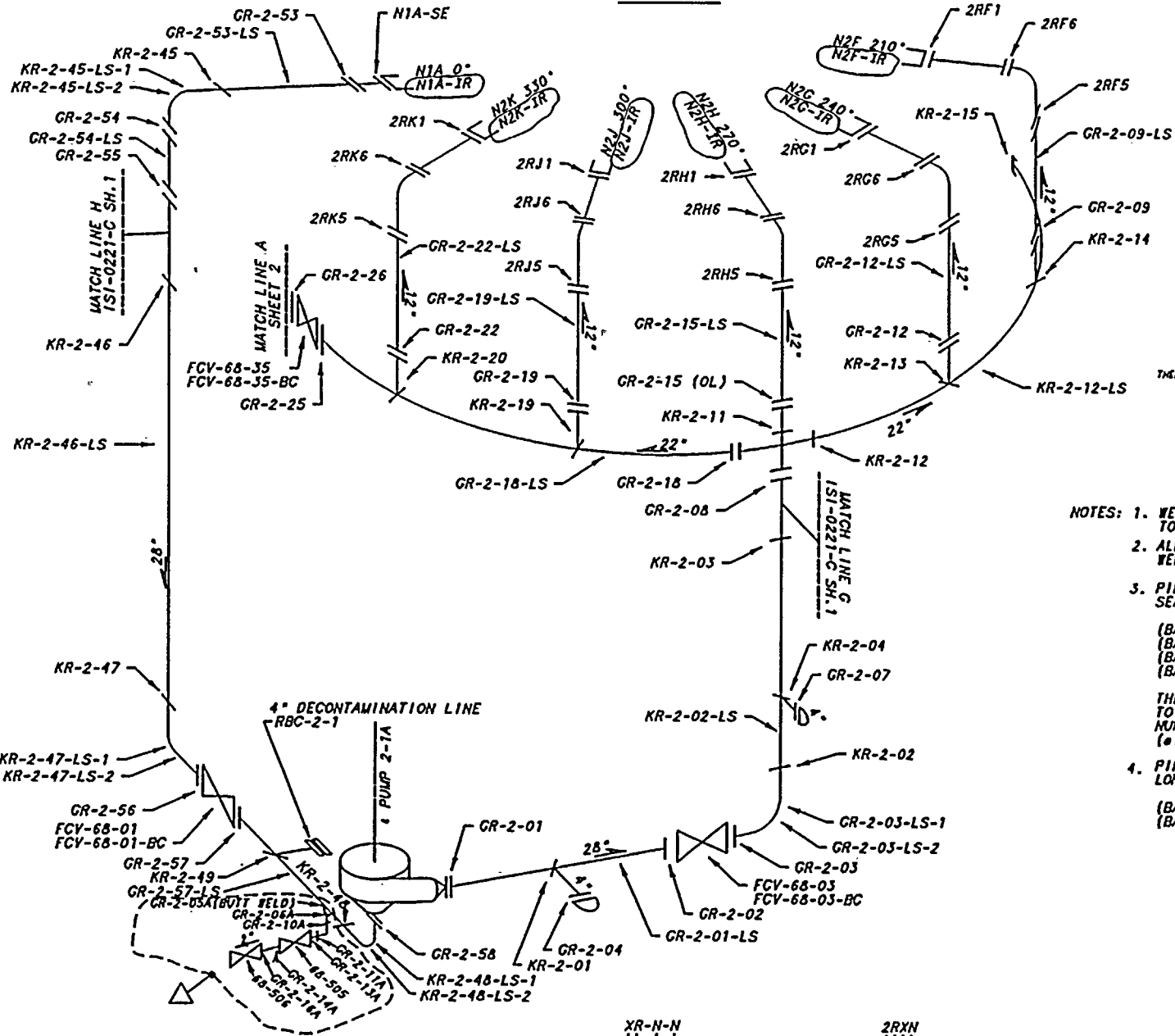


XFM-N-N
 K-KELLOGG SHOPWELD UNIT NO.
 G-GE FIELD WELD FEEDWATER WELD NO.

APPROV	REV.	DATE	BY	CHKD	ISSD	APPD	APPD	APPD	APPD
REVISED PER B110 SEC'D R14 870506 345									
CHANGE NO.	DATE	BY	CHKD	ISSD	APPD	APPD	APPD	APPD	APPD
TENNESSEE VALLEY AUTHORITY									
BRONNS FERRY NUCLEAR PLANT									
UNIT #2									
FEEDWATER SYSTEM									
WELD LOCATIONS									
DATE	BY	CHKD	ISSD	APPD	APPD	APPD	APPD	APPD	APPD
DATE	BY	CHKD	ISSD	APPD	APPD	APPD	APPD	APPD	APPD
DATE	BY	CHKD	ISSD	APPD	APPD	APPD	APPD	APPD	APPD
DATE	BY	CHKD	ISSD	APPD	APPD	APPD	APPD	APPD	APPD

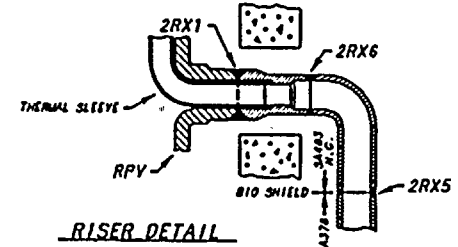


A - LOOP



MATERIAL SPECIFICATIONS:
 A358, TP 304
 4" X 0.337" NOM WALL THK. (SS)
 12" X 0.569" NOM. WALL THK. (SS)
 22" X 1.030" NOM. WALL THK. (SS)
 28" X 1.138" NOM. WALL THK. (SS) SUCTION
 28" X 1.322" NOM. WALL THK. (SS) DISCHARGE
 2" SCH. 80 A-376, TP304
 2" FITTINGS A-182, F304
 SAFE END REPLACEMENT
 12" X 0.688 NOM. WALL THK. (SS)
 SA 403 WP 316 N.G.

ASME CC-1 (EQUIVALENT)



- NOTES:
1. WELDS 2RX1 ARE THE NOZZLE TO SAFE-END WELDS
 2. ALL 2" WELDS ARE SOCKET WELDED EXCEPT WHERE NOTED.
 3. PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD NO.)-LS-1D (DOWNSTREAM)
 (BASE WELD NO.)-LS-2D (DOWNSTREAM)
 (BASE WELD NO.)-LS-1U (UPSTREAM)
 (BASE WELD NO.)-LS-2U (UPSTREAM)
 THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE, AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHERMOST FROM 0° ON THE PIPE. (e.g. -LS-1 at 130°, AND -LS-2 AT 310°)
 4. PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS
 (BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)

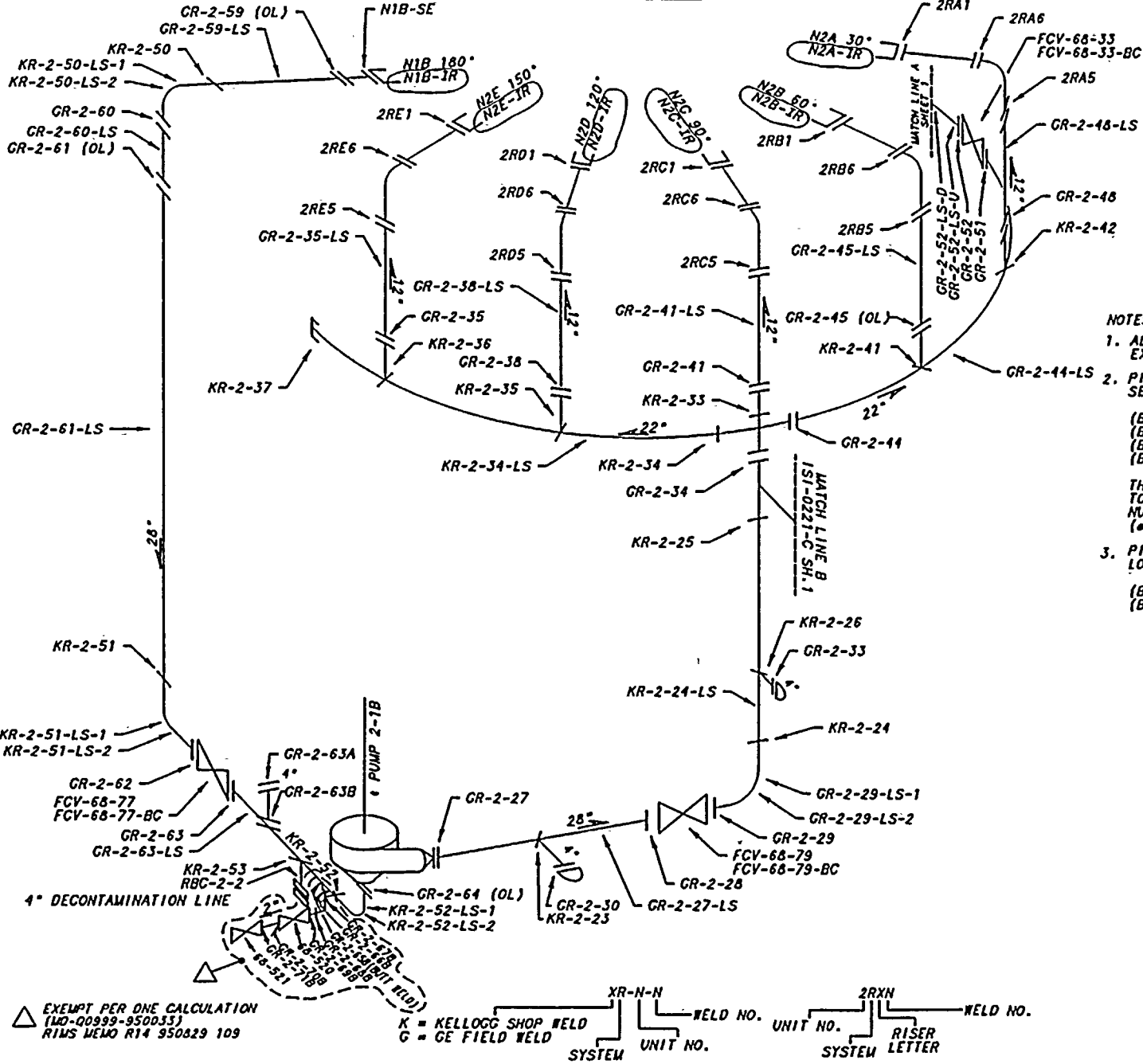
EXEMPT PER DNE CALCULATION
 (MD-00999-950033)
 RIMS MEMO R14 950829 109

K = KELLOGG SHOP WELD
 G = GE FIELD WELD
 WELD NO.
 UNIT NO.
 SYSTEM
 RISER LETTER

REV	DATE	BY	CHKD	APPD	DATE	BY	CHKD	APPD	DATE
1	08-14-88	EDC	GLB		08-14-88	EDC	GLB		08-14-88
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT									
UNIT 2									
RECIRCULATION SYSTEM									
WELD LOCATIONS									
DRAWN	PHD	SCALE	1:1	DATE	8-10-87	DATE	8-12-88	SHEET	1 OF 2 SHEETS
CHECKED	JIS	DATE	8-11-88	EDC	GLB	DATE	8-11-88	SCALE	1:1
2-151-0270-C(001)									



B - LOOP



- NOTES:**
- ALL 2" WELDS ARE SOCKET WELDED EXCEPT WHERE NOTED.
 - PIPE SEGMENTS CONTAINING TWO LONGITUDINAL SEAMS WILL BE IDENTIFIED AS:
 (BASE WELD NO.)-LS-1D (DOWNSTREAM)
 (BASE WELD NO.)-LS-2D (DOWNSTREAM)
 (BASE WELD NO.)-LS-1U (UPSTREAM)
 (BASE WELD NO.)-LS-2U (UPSTREAM)
 THE -LS-1 SEAM WILL BE NUMERICALLY CLOSEST TO 0° ON THE PIPE, AND THE -LS-2 SEAM WILL BE NUMERICALLY FARTHERMOST FROM 0° ON THE PIPE. (e.g. -LS-1 AT 130°, AND -LS-2 AT 310°)
 - PIPE SEGMENTS CONTAINING ONLY ONE LONGITUDINAL SEAM WILL BE IDENTIFIED AS
 (BASE WELD NO.)-LS-D (DOWNSTREAM)
 (BASE WELD NO.)-LS-U (UPSTREAM)

△ EXEMPT PER ONE CALCULATION
 (MD-00999-950033)
 RIMS MEMO R14 950029 109

K = KELLOGG SHOP WELD
 G = GE FIELD WELD

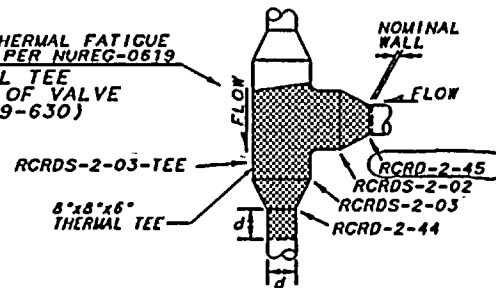
XR-N-N WELD NO.
 UNIT NO.

2RXN WELD NO.
 UNIT NO.
 RISER LETTER

ADMIN	REL.	R/A	R/A	R/A	R/A
REVISED PER RING WELD R14 950029 340					
CHANGE REF.	DATE	BY	CHKD	ESCH	BY
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 RECIRCULATION SYSTEM WELD LOCATIONS					
DRAWN: PHB	SUBMITTED	APPROVED	SCALE: NTS		
DATE: 3-28-89	DATE: 3-8-89	DATE: 2-17-89	SHEET 1 OF 2 SHEET(S)		
CHECKED: JLS	EDC	GLB	DRAWING NO.		
DATE: 3-3-89			2-151-0270-CDD2		
CCD					



REGION OF THERMAL FATIGUE
EXAMINATION PER NUREG-0619
THERMAL TEE
(UPSTREAM OF VALVE
2-FCV-69-630)



REFERENCE DRAWINGS
CRD-2-005
RCIC-2-004
RWC-2-001
47X335-14, -17

NOTE:
THIS DRAWING SUPERSEDES CHM-2075-C
AND CHM-2072-C (ALL SHEETS)

MATERIAL SPECIFICATIONS

STAINLESS STEEL

FITTINGS

6" SA403 WP316NG SCH. 80 SS

PIPING

6" SA376 TP316NG SCH. 80 SS

CARBON STEEL

4" SCH. 80 A-333, GRI (SEAMLESS) CS

6" X 0.562" NOM WALL SCH. 120 CS

8" X 0.593" NOM WALL SCH. 100 CS

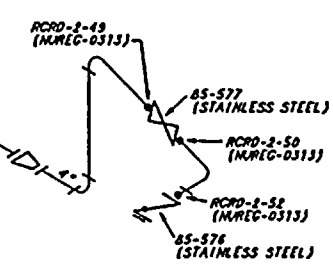
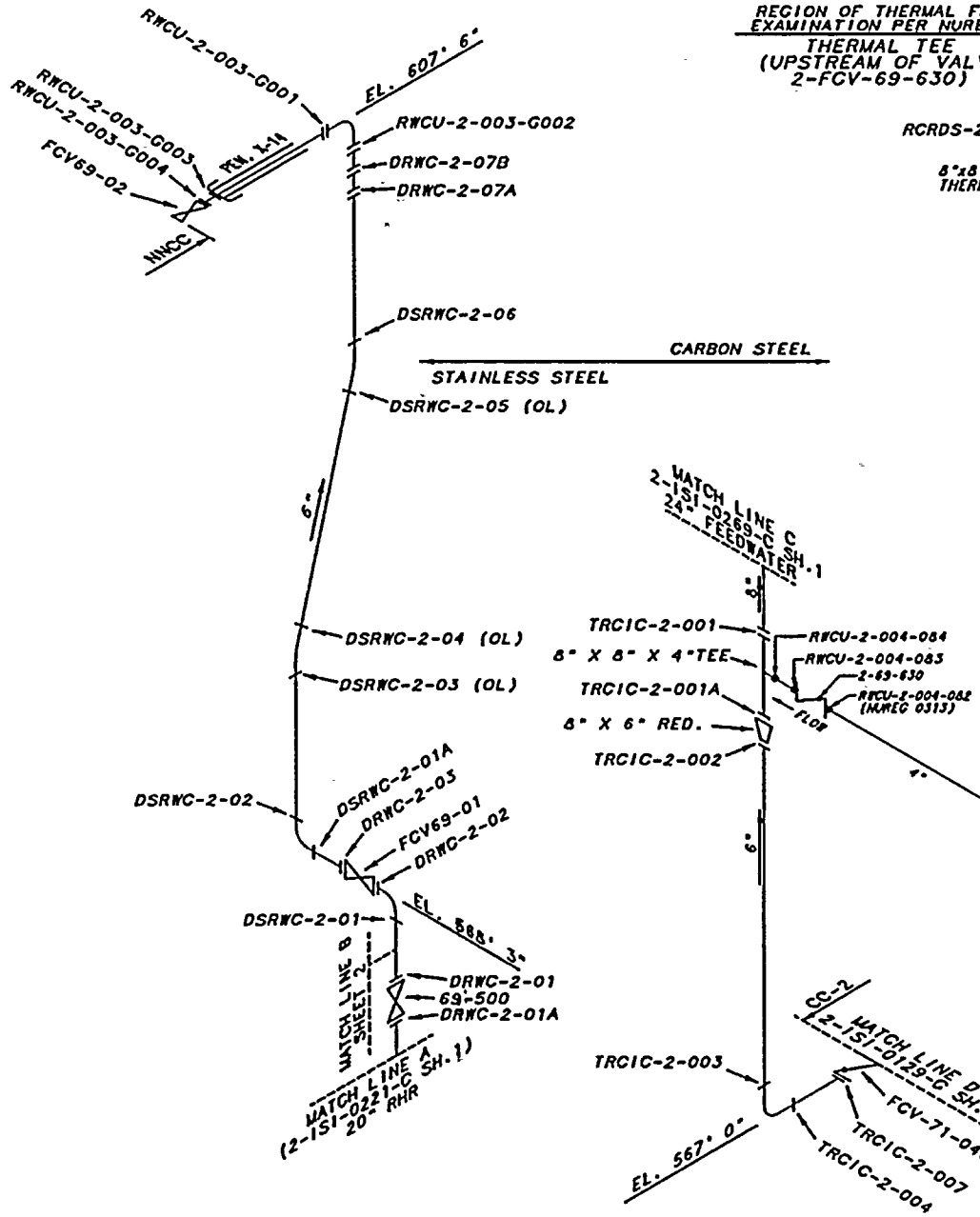
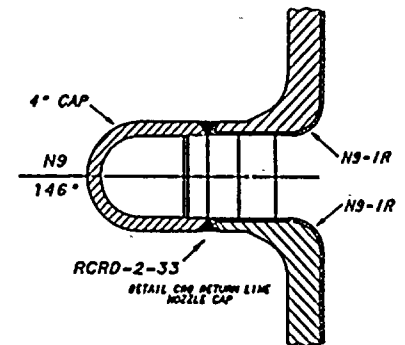
VALVE

2-69-630 SA162 F316

CRD CAP

4" X 0.674" NOM WALL SS

ASME CC-1 (EQUIVALENT)

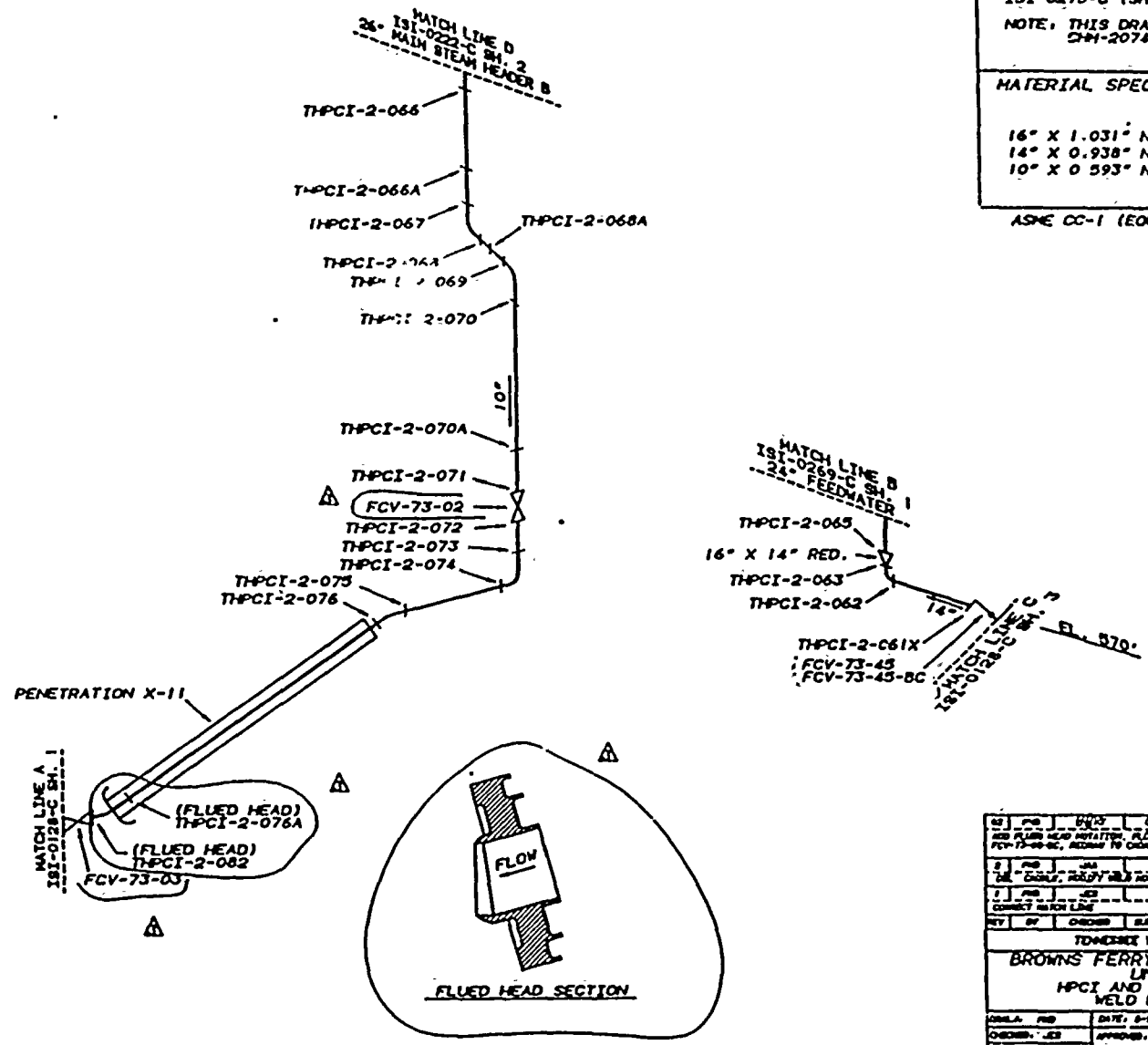


004	JHW	DDP	JT	RLB	3-5-84
REVISED PER RINE MEMO R21 948263 001 (ADMINISTRATIVE REVISION)					
REV	CHANGE REF	PREPARED	CHECKER	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 2 REACTOR WATER CLEAN UP, RCIC, AND CRD WELD IDENTIFICATION					
DRAWN: PWB	DATE: 8-9-88	SCALE: NTS	EXAM/ISS/IMP		
CHECKED: JES	APPROVED:	SHEET 01 OF 03		REV	
SUBMITTED: EDC	QUB	2-151-0272-C-001			
CCD					



REFERENCE DRAWINGS
 TVA 47N335-11
 TVA 47N335-12
 TVA 47N1947
 ISI-0275-C (SH. 1) SUPPORT MAP
 NOTE: THIS DRAWING SUPERSEDES
 CH-2074-C ALL SHEETS

MATERIAL SPECIFICATIONS
 16" X 1.031" NOM. WALL THK. CS
 14" X 0.938" NOM. WALL THK. CS
 10" X 0.593" NOM. WALL THK. CS
 ASME CC-1 (EQUIVALENT).

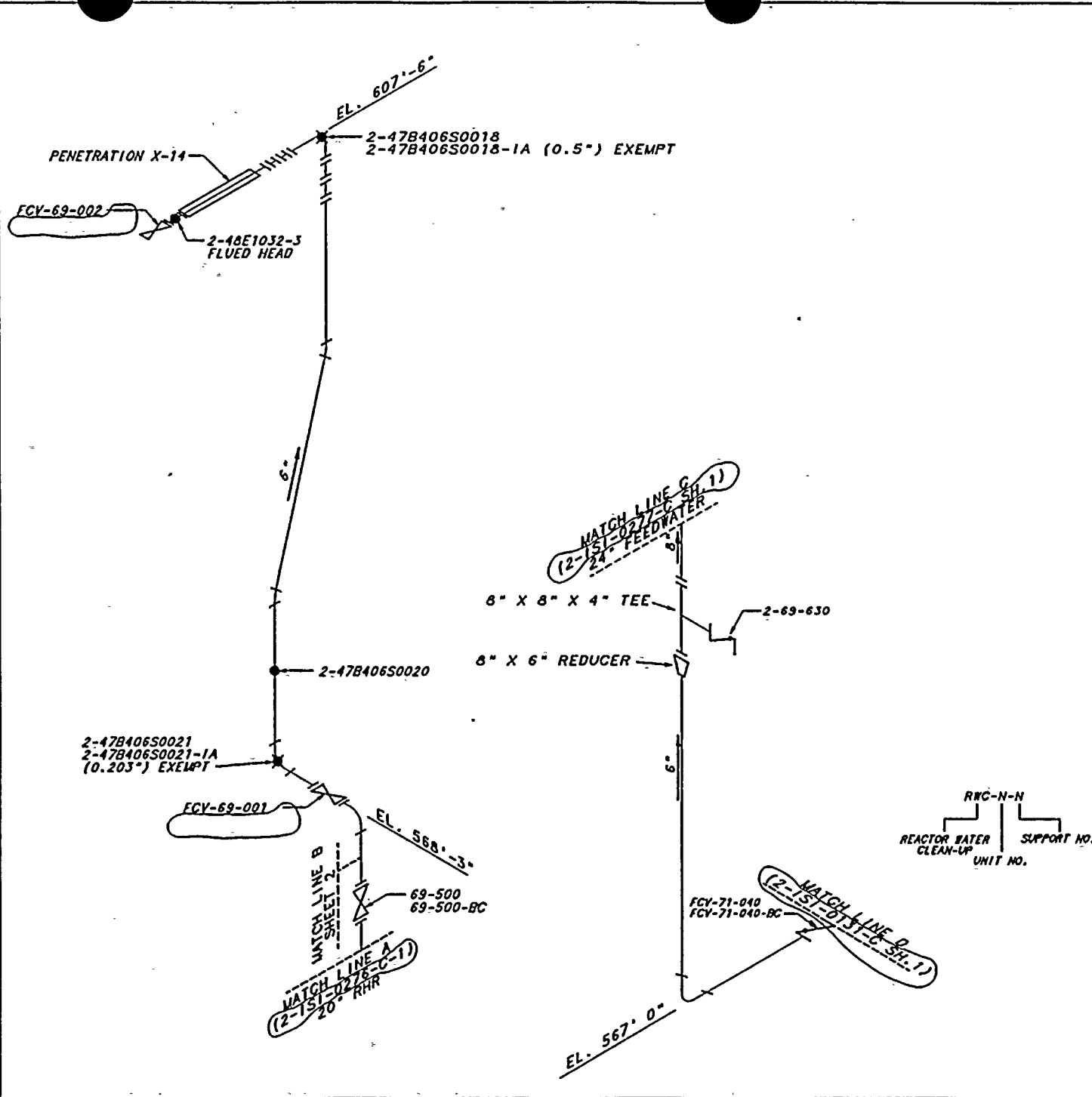


NO.	REV.	DATE	BY	CHKD.	APP'D.
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2	002
3	003
4	004
5	005
6	006
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8	008
9	009
10	010
11	011
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13	013
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15	015
16	016
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18	018
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100	100

TENNESSEE VALLEY AUTHORITY
 BROWNS FERRY NUCLEAR PLANT
 UNIT 2
 HPCI AND RWCU SYSTEMS
 WELD LOCATIONS

DATE: 8-2-88
 SHEET 01 OF 01
 ISI-0273-C-03





REFERENCE DRAWING
 TVA 47W335-14
 TVA 47W335-17
 TVA 47K177A
 2-151-0272-C (SH. 1) WELD MAP

NOTE:
 THIS DRAWING SUPERSEDES CHM-2082-C
 (ALL SHEETS)

LEGEND:
 ■ VARIABLE SUPPORT
 ● RIGID HANGER
 ○ HYDRAULIC SNUBBER
 — RIGID STRUT

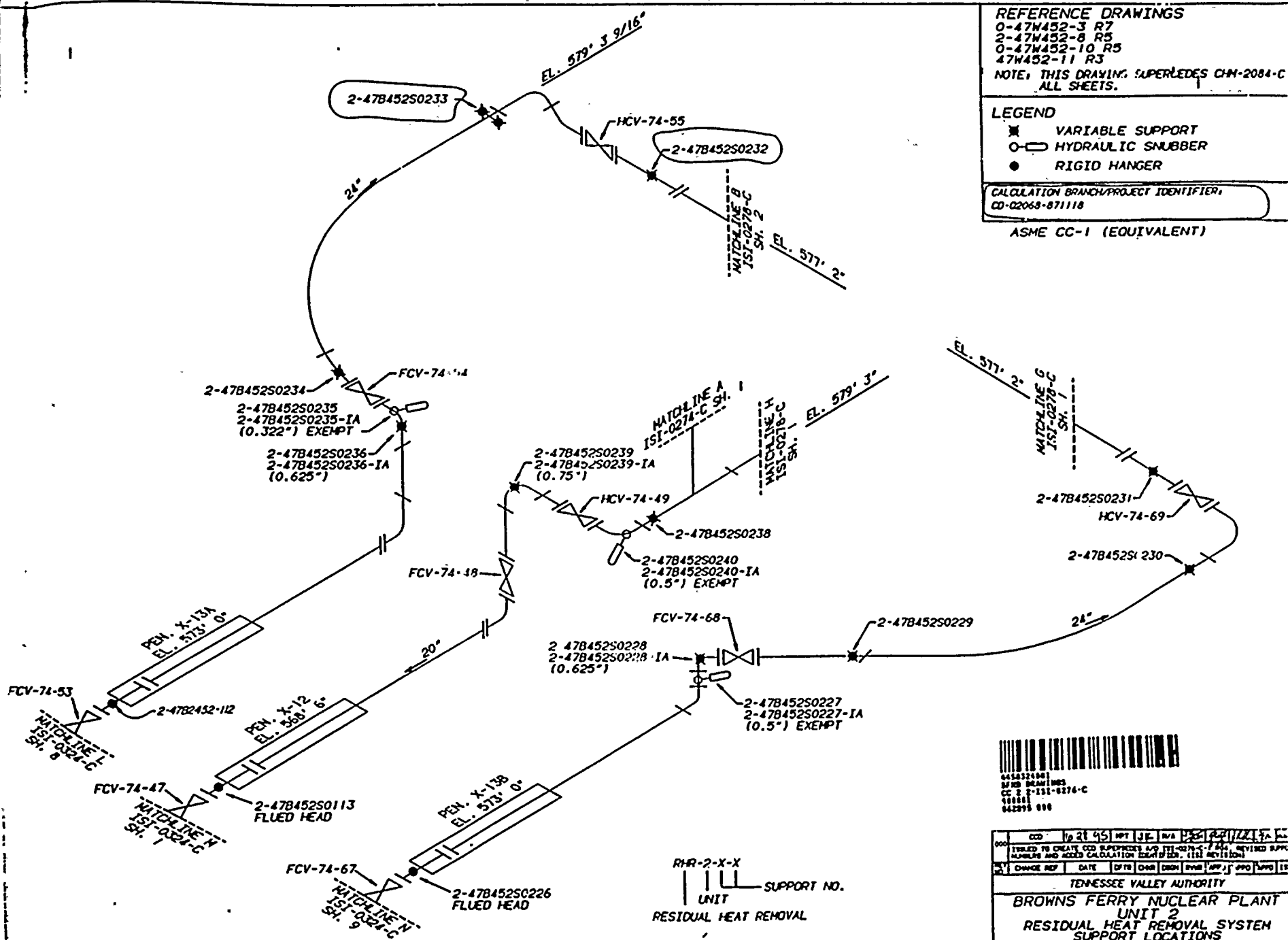
CALCULATION BRANCH/PROJECT IDENTIFIER:
 CD-02069 - 300011
 CD-02071 - 880988

ASME CC-1 (EQUIVALENT)



001	ADMIN	RDL	N/A	N/A	N/A	N/A	N/A	N/A	N/A
	REVISED PER RIVS MEMO R14 870503 307 AND REFERENCE DRAWINGS								
001	CHANGE REF	DATE	CFR	DRR	DSCH	ENVR	APPO	APPO	ISSO
TENNESSEE VALLEY AUTHORITY									
S	BROWNS FERRY NUCLEAR PLANT UNIT 2 REACTOR WATER CLEAN-UP, RCIC, AND CRD HYDRAULIC RETURN LINE, SUPPORT IDENTIFICATION								
DRAWN:	PHB	DATE:	8-13-88	SCALE:	NIS	CDRAWN:	JSC		
CHECKED:	JES	APPROVED:							
SUBMITTED:	EDC	CLB							
								2-151-0274-C-001	
CCD									



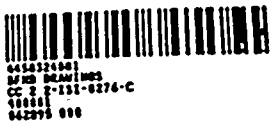


REFERENCE DRAWINGS
 0-47W452-3 R7
 2-47W452-8 R5
 0-47W452-10 R5
 47W452-11 R3
 NOTE: THIS DRAWING SUPERSEDES CHN-2084-C ALL SHEETS.

LEGEND
 [Square symbol] VARIABLE SUPPORT
 [Circle symbol] HYDRAULIC SNUBBER
 [Circle symbol] RIGID HANGER

CALCULATION BRANCH/PROJECT IDENTIFIER:
 CD-02063-871118

ASME CC-1 (EQUIVALENT)



CD	10/21/95	REV	3	BY	PSL	APP	DATE	CHKD	DATE	APP	DATE
TENSESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS											
CHNG	NO.	DATE	BY	CHKD	DATE	APP	DATE	CHKD	DATE	APP	DATE
DRAFT: 10-20-95 CHECKED: 10-20-95 APPROVED: 10-20-95 ISSUE NO. 01 TOTAL NO. 01											



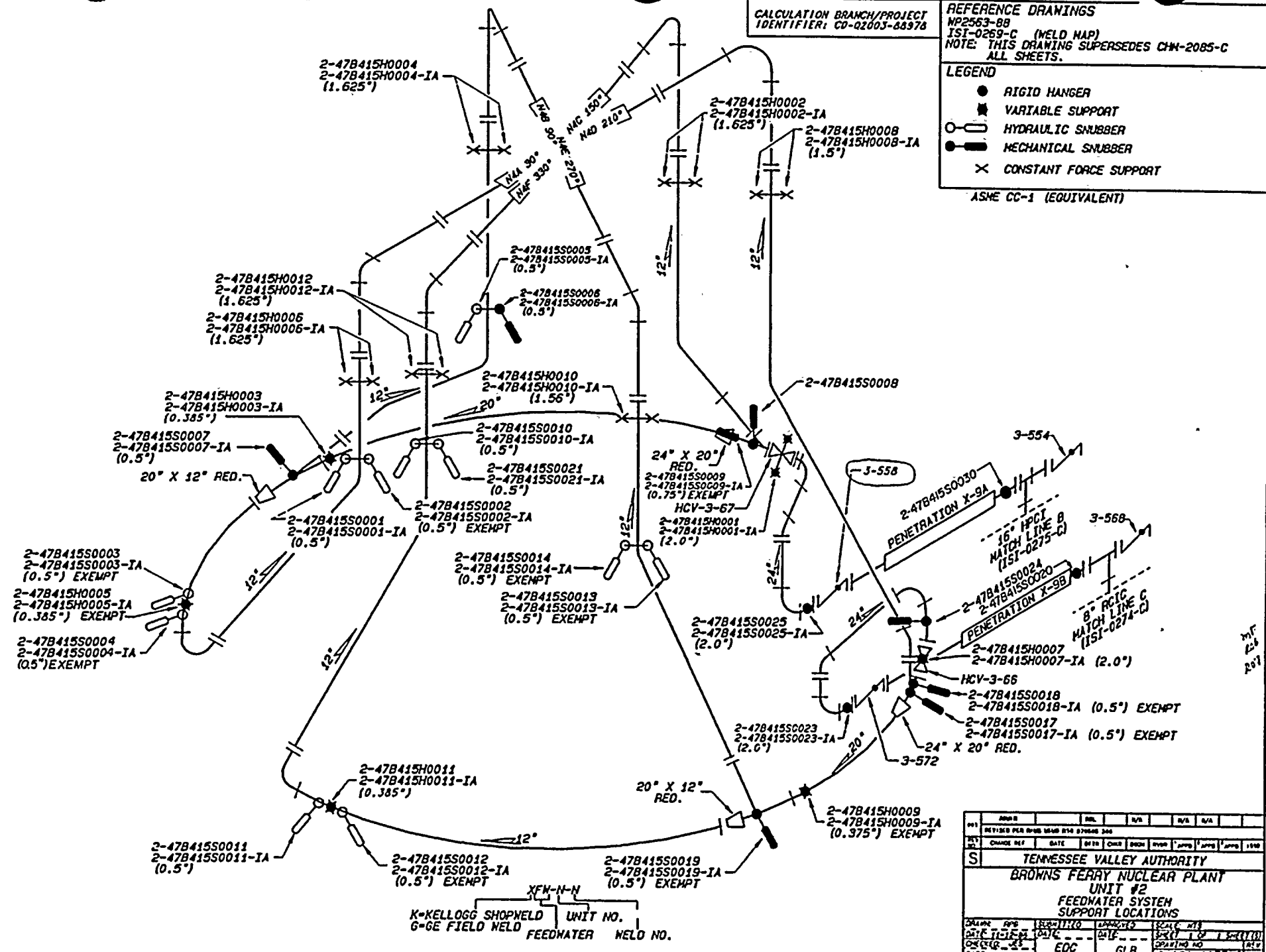
CALCULATION BRANCH/PROJECT
IDENTIFIER: CD-Q2003-88978

REFERENCE DRAWINGS
NP2563-88
ISI-0269-C (WELD MAP)
NOTE: THIS DRAWING SUPERSEDES CHN-2085-C
ALL SHEETS.

LEGEND

- RIGID HANGER
- ★ VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- MECHANICAL SNUBBER
- × CONSTANT FORCE SUPPORT

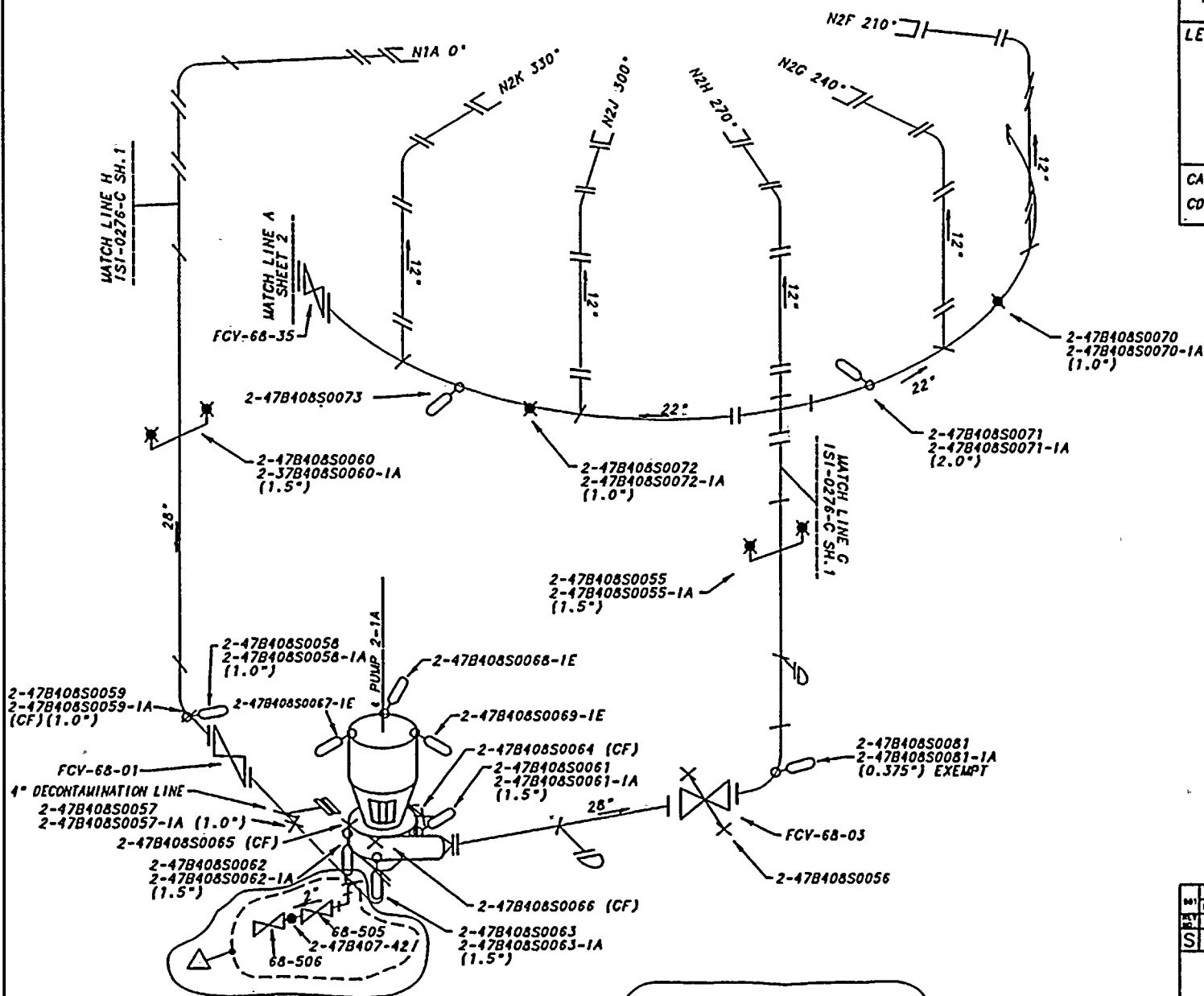
ASME CC-1 (EQUIVALENT)



NO.	DATE	BY	CHKD	APPD	REV	DATE	BY	CHKD	APPD
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A - LOOP



REFERENCE DRAWINGS:
 2-47B2408-8,9 (S.E. ACCEMENT)
 GE 769E963 (S.E. REPLACEMENT)
 TYA 47K1544-2
 2-153F754
 KELLOGG BF 2-180
 NOTE: THIS DRAWING SUPERSEDES
 CHM-2068-C ALL SHEETS

LEGEND:
 ● RIGID HANGER
 ✕ VARIABLE SUPPORT
 ○ HYDRAULIC SNUBBER
 ✕ CONSTANT FORCE SUPPORT (CF)

CALCULATION BRANCH/PROJECT IDENTIFIER:
 CD-Q2068-871118

ASME CC-1 (EQUIVALENT)

NO	APPRO	REV	R/A	R/A	R/A
1	REVISED PER RIMS MEMO R14 950829 347				
2	CHANGE REF	DATE	BY (R)	CHKD	DESCR
S	TENNESSEE VALLEY AUTHORITY				
BROWNS FERRY NUCLEAR PLANT UNIT 2 RECIRCULATION SYSTEM SUPPORT LOCATIONS					
DRAWN: PFB	SUBMITTED	APPROVED	SCALE: NTS		
DATE: 9-22-87	DATE: 8-18-88	DATE: 12-11-88	SHEET 1 OF 2 SHEETS		
CHECKED: JLS	EDC	GLB	SHEET NO. 112		
DATE: 9-10-87			2-151-0278-C-001		
			CCD		



CLCUALTION BRANCH/PROJECT IDENTIFIER: CD-02068-871118

REFERENCE DRAWINGS:

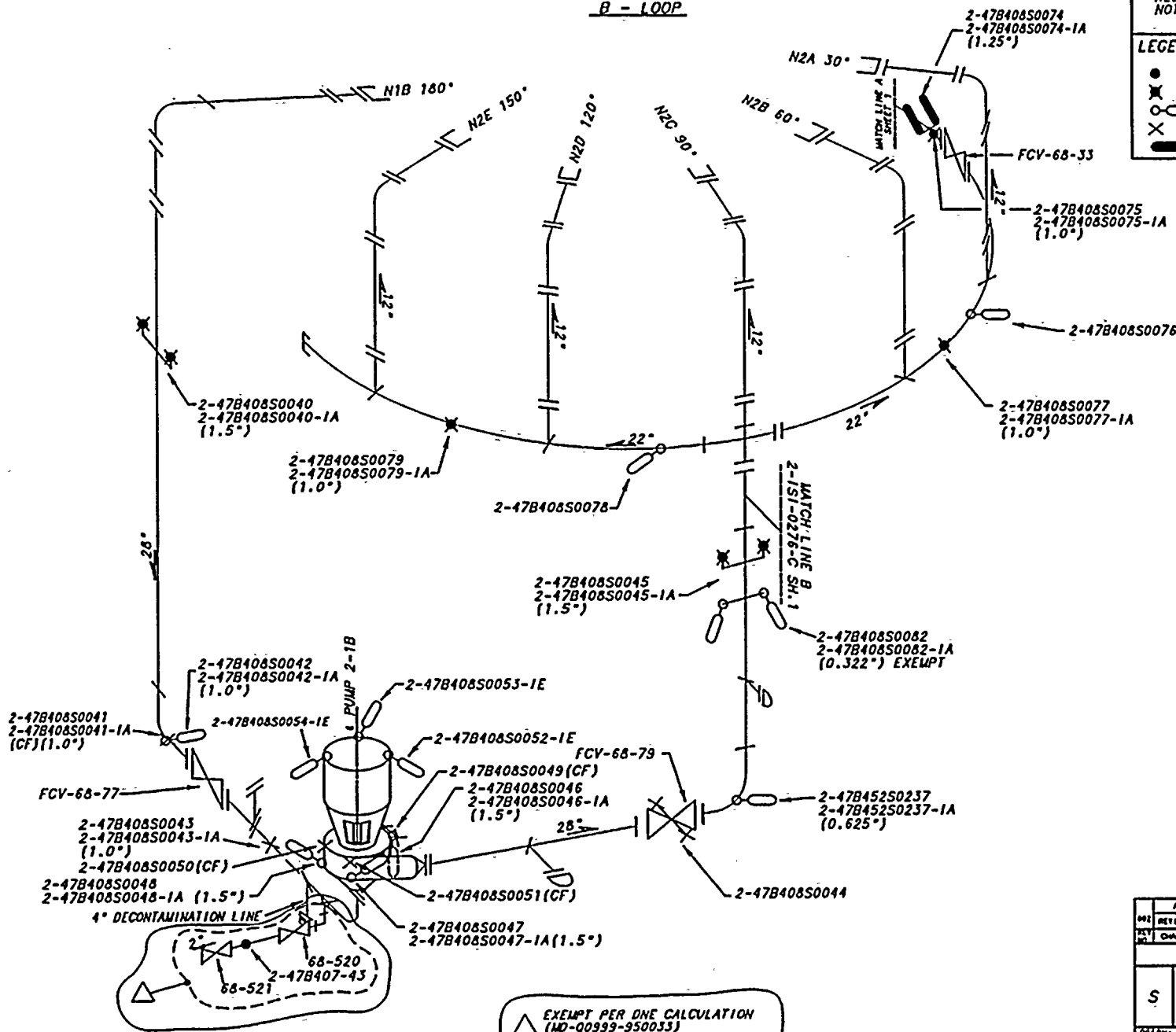
TVA 47K1544-2
 GE 2-153F754
 KELLOGG BF 2-180
 NOTE: THIS DRAWING SUPERSEDES
 CHM-2068-C ALL SHEETS

LEGEND:

- RIGID HANGER
- ⊗ VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- ⊗ CONSTANT FORCE SUPPORT (CF)
- ▬ RIGID STRUT

ASME CC-1 (EQUIVALENT)

B - LOOP



△ EXEMPT PER DNE CALCULATION
 (MD-00999-950033)
 RIMS MEMO R14 950829 109

NO.	DESCRIPTION	DATE	BY	CHKD	ISSD	APPV	APPV	APPV	APPV	1530	
001	REVISED PER RIMS MEMO R14 950829 109										
002	CHANGE REF										
TENNESSEE VALLEY AUTHORITY											
BROWNS FERRY NUCLEAR PLANT UNIT 2 RECIRCULATION SYSTEM SUPPORT LOCATIONS											
DATE:	3-28-89	SCALE:	1/2" = 1'-0"	SCALE NO.:	2-151-0278-C	REV.:	002				
DESIGNER:	EDC	APPROVED:	GLB	ISSUING NO.:	2-151-0278-C	REV.:	002				
DATE:	3-8-89										CCD



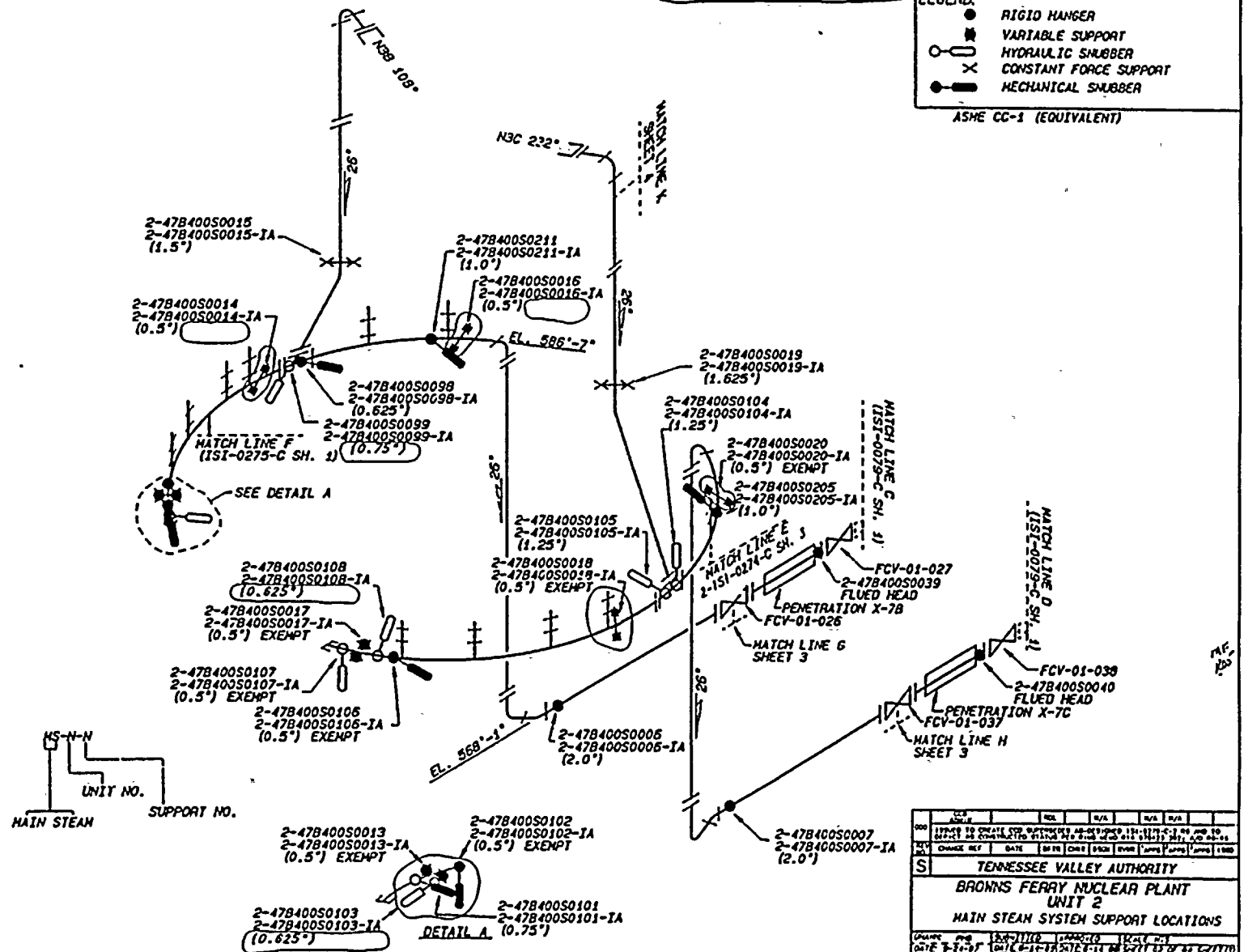


CALCULATION BRANCH/PROJECT
IDENTIFIERS: CD-02001-88972
CD-02001-88970

REFERENCE DRAWINGS:
GE 729E401 SH. 1 & 2
ISI-0222-C SH. 2 (NELO MAP)
NOTE: THIS DRAWING SUPERCEDES CHN-2087-C
SH. 2

LEGEND:
● RIGID HANGER
⊕ VARIABLE SUPPORT
○ HYDRAULIC SNUBBER
× CONSTANT FORCE SUPPORT
● MECHANICAL SNUBBER

ASME CC-1 (EQUIVALENT)



REV	DATE	BY	CHKD	APP'D	REASON
001	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
002	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
003	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
004	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
005	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
006	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
007	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
008	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
009	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
010	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
011	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
012	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
013	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
014	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
015	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
016	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
017	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
018	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
019	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION
020	11/11/10	CD	GLB	EDC	ISSUED FOR CONSTRUCTION

TENNESSEE VALLEY AUTHORITY
BRONNS FERRY NUCLEAR PLANT
UNIT 2
MAIN STEAM SYSTEM SUPPORT LOCATIONS

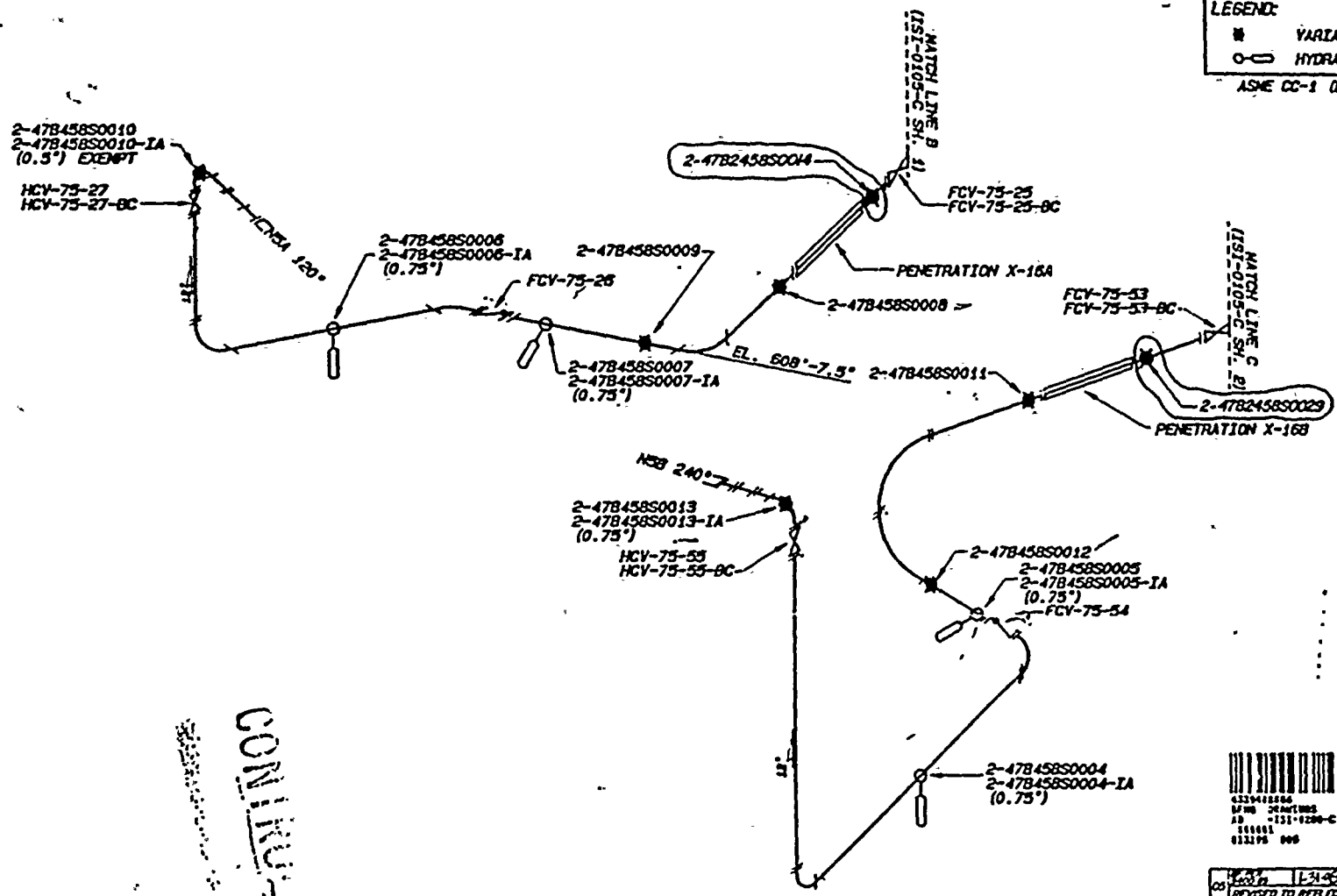
DATE: 11/11/10
BY: CD
CHKD: GLB
APP'D: EDC

CD

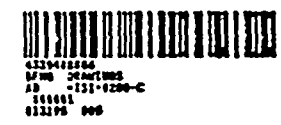
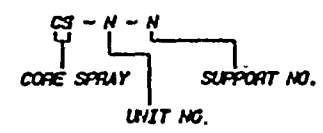


REFERENCE DRAWINGS:
 47N458-198
 ISI-0271-C (SH. 1) WELD MAP
 NOTE: THIS DRAWING SUPERCEDES CHN-2009-C
 ALL SHEETS.

LEGEND:
 ■ VARIABLE SUPPORT
 ○-○ HYDRAULIC STRUT
 ASME CC-1 (EQUIVALENT)



CONTINUED



DATE	BY	REV	DESCRIPTION	APP'D	SCALE
01/15/88	JL	1	REVISED TO REFLECT REVIEW OF PIP ST. DISTANCES		
01/15/88	JL	2	REVISE SUPPORT NUMBERS & MATCH LINE NUMBERS FOR LATEST UADS		
01/15/88	JL	3	REVISE SUPPORT NUMBERS & MATCH LINE NUMBERS FOR LATEST UADS		
01/15/88	JL	4	ADD NEW SUPPORT T'S		
01/15/88	JL	5	ADD MATCH LINE T'S		
01/15/88	JL	6	REVISION		
01/15/88	JL	7	REVISION		

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT
UNIT 2
CORE SPRAY SYSTEM
SUPPORT LOCATIONS

DATE	BY	REV	DESCRIPTION	APP'D	SCALE
01/15/88	JL	1	REVISED TO REFLECT REVIEW OF PIP ST. DISTANCES		
01/15/88	JL	2	REVISE SUPPORT NUMBERS & MATCH LINE NUMBERS FOR LATEST UADS		
01/15/88	JL	3	REVISE SUPPORT NUMBERS & MATCH LINE NUMBERS FOR LATEST UADS		
01/15/88	JL	4	ADD NEW SUPPORT T'S		
01/15/88	JL	5	ADD MATCH LINE T'S		
01/15/88	JL	6	REVISION		
01/15/88	JL	7	REVISION		

EDC GLB
 151-0200-C

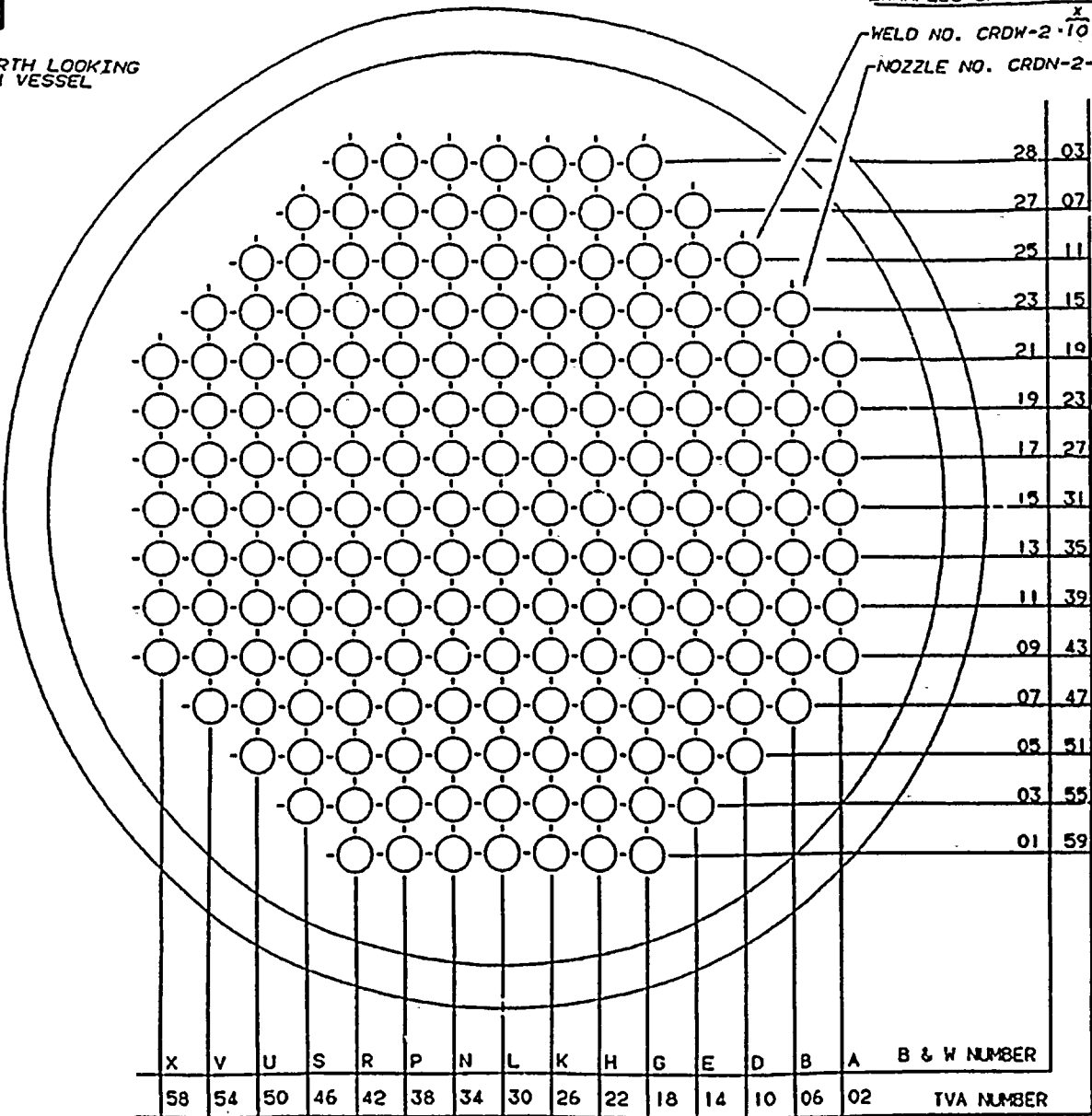




ASSUMED NORTH LOOKING
DOWN ON VESSEL

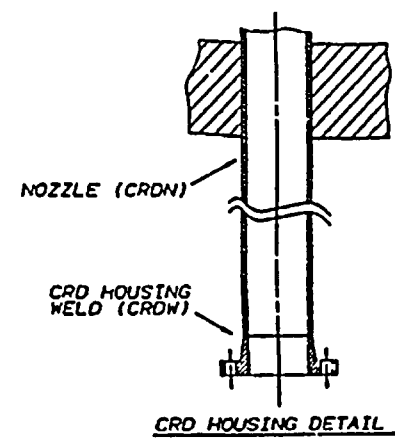
EXAMPLES OF TVA NUMBERS

WELD NO. CRDN-2-^X1011^Y
NOZZLE NO. CRDN-2-0615



NOTES:

1. THIS DRAWING SUPERSEDES CHM-2002-C (UNIT 2)
2. NOZZLES ARE SPECIFIED BY CRDN-2-"X"-Y"
3. WELDS ARE SPECIFIED BY CRDN-2-"X"-Y"
4. BOLTING IS SPECIFIED BY CRDN-2-"X"-Y"-BC.



X	V	U	S	R	P	N	L	K	H	G	E	D	B	A	B & W NUMBER
58	54	50	46	42	38	34	30	26	22	18	14	10	06	02	TVA NUMBER

X

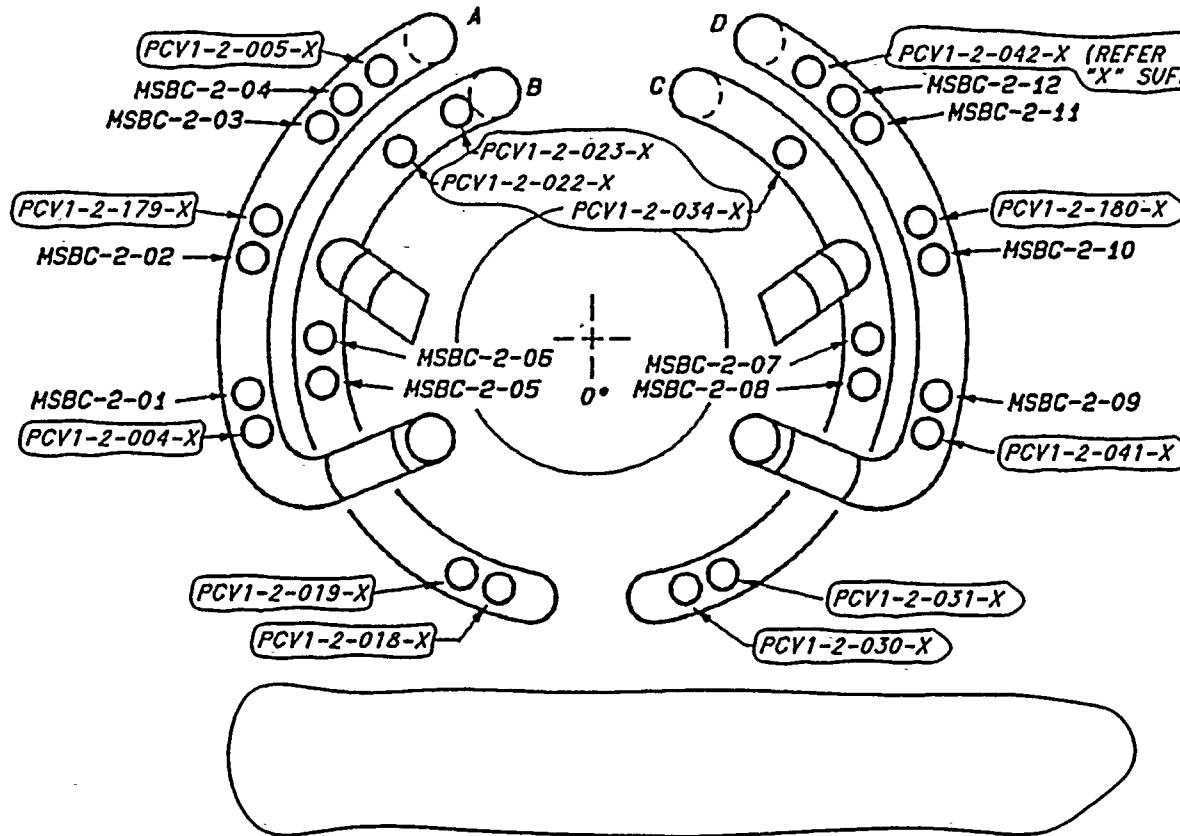
APP'D	CHK'D	IN'G	DATE	BY	DATE	BY	DATE	BY	DATE	BY	DATE	BY	DATE	BY	DATE	BY	DATE	BY
REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214	REVISED FOR BUREAU 214
DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82	DATE: 7-17-82
CREATED: 4-82	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81	DATE: 3-7-81
DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81
TENNESSEE VALLEY AUTHORITY																		
BROWNS FERRY NUCLEAR PLANT																		
UNIT 2																		
S REACTOR VESSEL PENETRATIONS																		
CONTROL ROD DRIVE																		
SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"	SCALE: 1/4" = 1'-0"
DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81	DATE: 8-7-81
CCD																		



REFERENCE DRAWINGS:

2-47E801-1

ASME CC-1 (EQUIVALENT)



ALL REVISION CIRCLES
ARE ADMINISTRATIVE REVISION

NOTES:

1. THIS DRAWING SUPERSEDES ISI-0027-B FOR UNIT 2 ONLY.
2. SUBSTITUTE "YBC" FOR "X" WHEN EXAMINING VALVE BODY BOLTING AND "PBC" FOR "X" WHEN EXAMINING VALVE TO PIPE BOLTING.

HARDWARE	TEKTRONIX 4129
SOFTWARE	TEKNICAD 8.2
FLOPPY OR TAPE #	FLOPPY #BF4

* FOR ORIGINAL SIGNATURES AND PREVIOUS REV. INFO. SEE "ORIGINAL".

PROCAD

ALL A/D HISTORY RESEARCHED AT R000

CCD	ADMIN	BCC	N/A	N/A	N/A				
000	ISSUED TO CREATE CCD, SUPERSEDES AS-DESIGNED ISI-0312-B-1 R2 AND TO DEPICT AS-CONSTRUCTED STATUS PER A/D R0, R1; RIUS MEWO R32 360604 850 (ADMINISTRATIVE REVISION)								
REV #	CHANGE REF	DATE	CFR	CHK	DSGN	RYER	APPD	APPD	ISSD
TENNESSEE VALLEY AUTHORITY									
S	BROWNS FERRY NUCLEAR PLANT UNIT 2 MAINSTEAM SYSTEM BOLTED CONNECTIONS								
DRAWN	PHB	SUBMITTED	APPROVED	SCALE	NTS				
DATE	#	DATE	DATE	SHEET	1 OF 1 SHEET(S)				
CHECKED	#	*	*	DRAWING NO.	2-ISI-0312-B				
DATE	#			REV	000				
									CCD

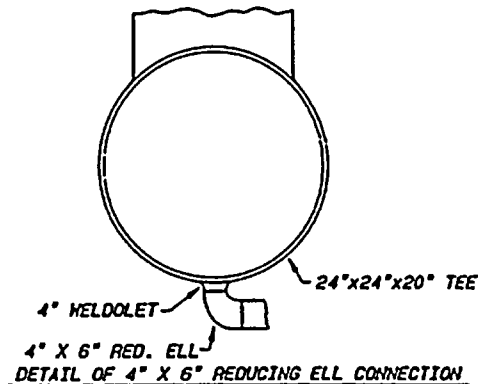
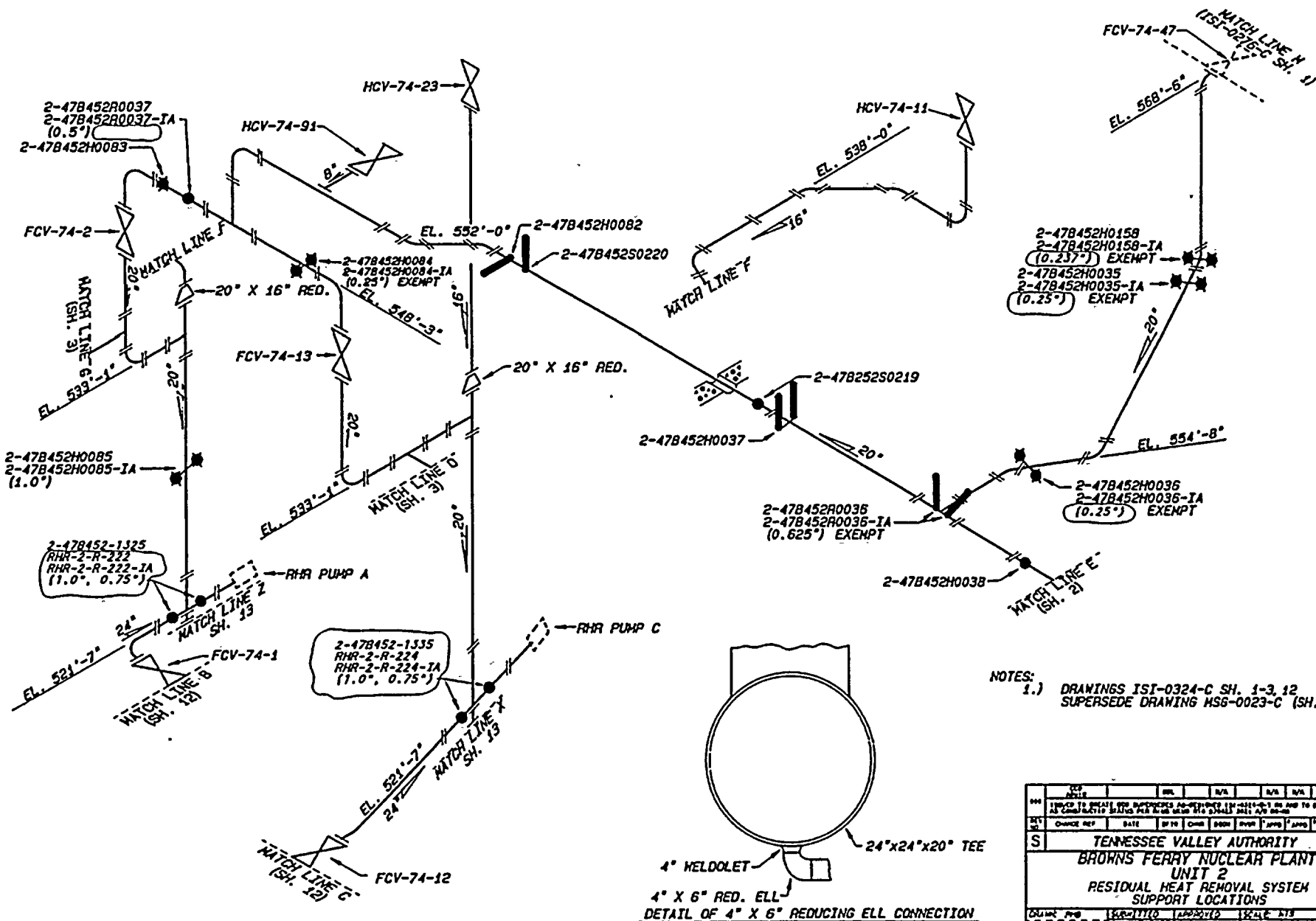


CALCULATION BRANCH/PROJECT IDENTIFIER: CD-Q2073-883.

LEGEND:
 RIGID HANGER
 VARIABLE SUPPORT
 RIGID STRUT

ASME CC-2 (EQUIVALENT)

REFERENCE DRAWINGS:
 47N452 SERIES
 47N335-4
 MS6-0018-C (SH. 1) WELD MAP
 THIS DRAWING SUPERSEDES MS6-0023-C SH. (SEE NOTE 1)



NOTES:
 1.) DRAWINGS ISI-0324-C SH. 1-3, 12
 SUPERSEDE DRAWING MS6-0023-C (SH. 1)

REV	NO	DATE	BY	CHKD	APP'D	APP'D	APP'D	APP'D	APP'D
1									

TENNESSEE VALLEY AUTHORITY
 BRORNS FERRY NUCLEAR PLANT
 UNIT 2
 RESIDUAL HEAT REMOVAL SYSTEM
 SUPPORT LOCATIONS

DATE: 6-2-88	DATE: 6-2-88	DATE: 6-2-88	DATE: 6-2-88
CD: GCP-05	EDC	GLB	2-151-0324-C 000
DATE: 6-2-88			

CCD



NOTES
1

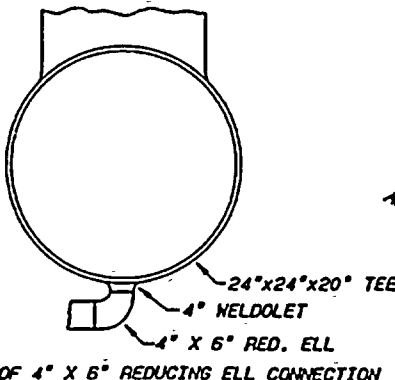
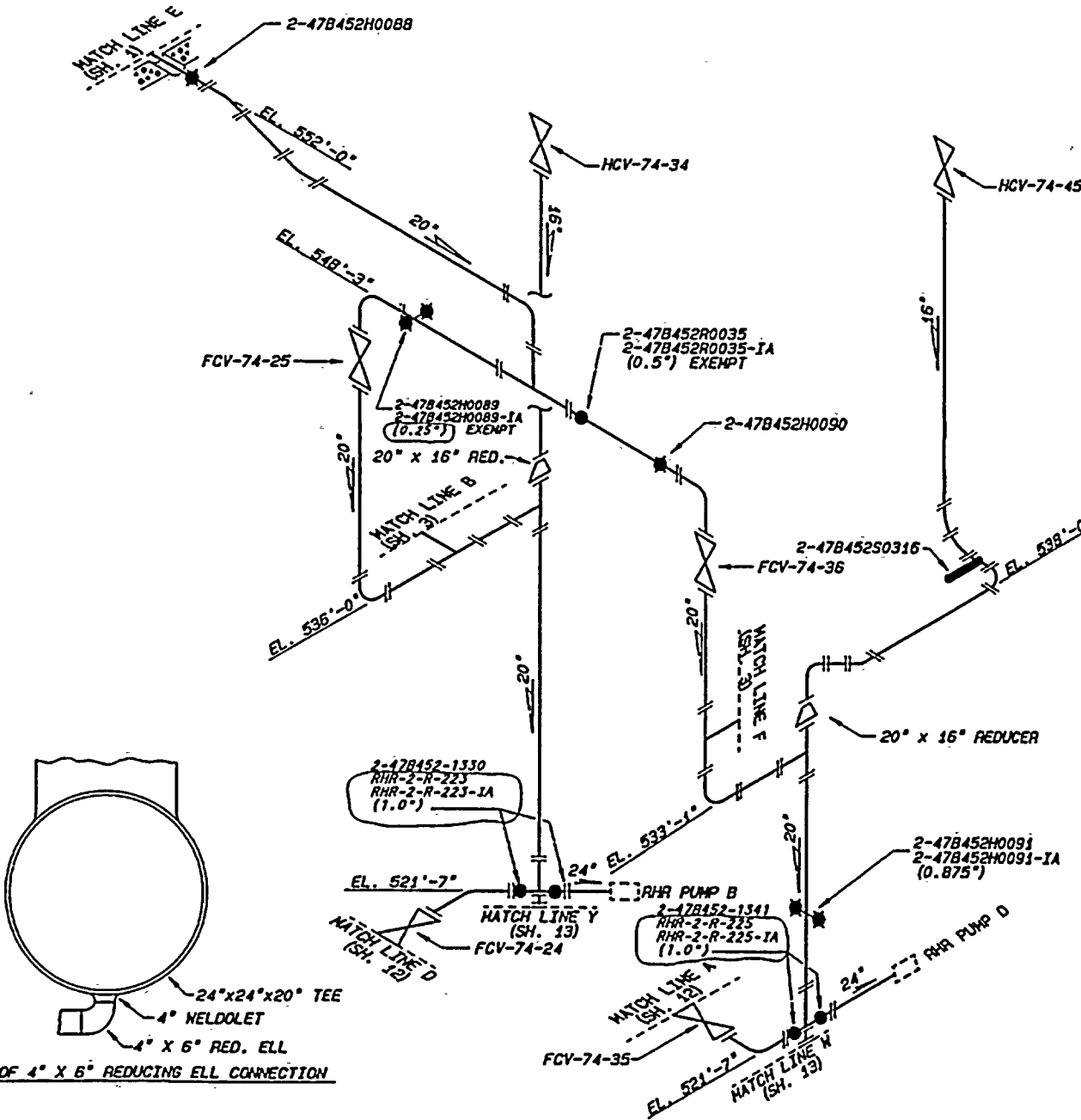
DRAWINGS ISI-0324-C SH. 1-3, 12 MAKE UP
SUPERCEDED DRAWING MSG-0023-C (SH. 1)

REFERENCE DRAWINGS
47N452 SERIES
47N335-4
MSG-0018-C (SH. 2)
THIS DRAWING SUPERCEDES MSG-0023-C SH. 1
(SEE NOTE 1)

LEGEND
● RIGID SUPPORT
⊗ VARIABLE SUPPORT
— RIGID STRUT

CALCULATION BRANCH/PROJECT
IDENTIFIER: CD-Q207J-663012

ASME CC-2 (EQUIVALENT)



DETAIL OF 4\"/>

NO	DATE	BY	CHKD	APPD	REVISION
1	6-8-88	EDC	GLB		ISSUED FOR CONSTRUCTION
2	6-8-88	EDC	GLB		CHANGE REF

TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
SUPPORT LOCATIONS					
SCALE: 1/8\"/>					



NOTES:

1. DRAWINGS ISI-0324-C CH. 8 AND 9 MAKE L SUPERSEDED DRAWING MSG-0023-C (SH. 3)
2. POSSIBLE EXCESSIVE GRIND SPOT BELOW RHR-2-H-7 (RS) @ 7.00
3. DRAWING NUMBERS PROVIDED IN PARENTHESIS. DRAWING MAY HAVE MULTIPLE SHEETS.

CALCULATION BRANCH/PROJECT IDENTIFIER, CO-02074-880991
CO-02074-870644

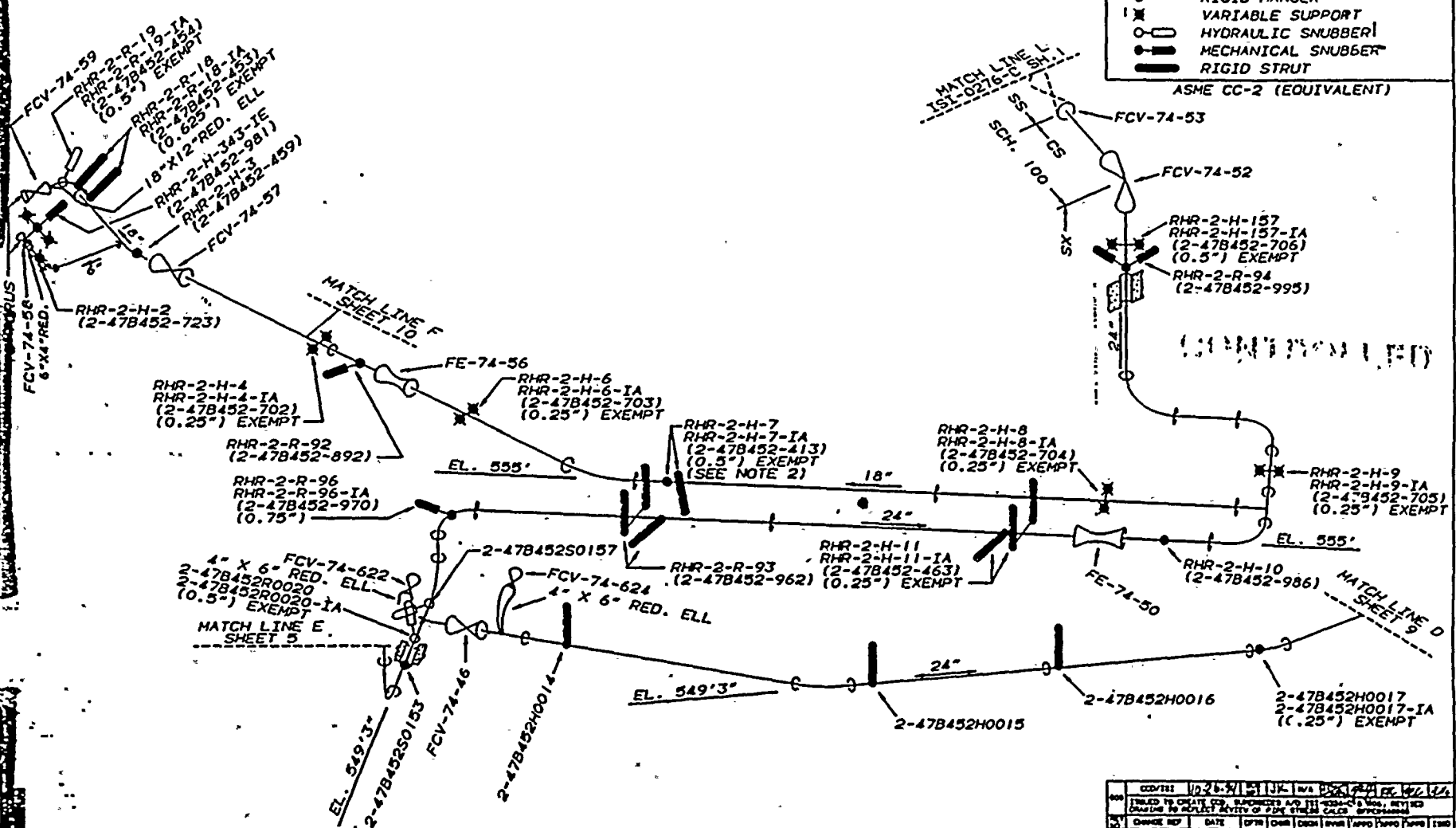
REFERENCE DRAWING
47H452-221
47H335-6
WP2563-88
MSG-0018-C (SH.8)) MAP

THIS DRAWING SUPERSEDES MSG-0023-C SH.3 (SEE NOTE 1)

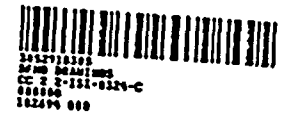
LEGEND

- RIGID HANGER
- ⊗ VARIABLE SUPPORT
- HYDRAULIC SNUBBER
- ⊖ MECHANICAL SNUBBER
- RIGID STRUT

ASME GC-2 (EQUIVALENT)



ALL REVISION CIRCLES ARE OMITTED FOR CLARITY








CCD/ISS		10/26/94		10/26/94		10/26/94		10/26/94		10/26/94	
NO	ISSUED BY	DATE	DESCRIPTION	APPROVED	DATE	ISSUED BY	DATE	DESCRIPTION	APPROVED	DATE	ISSUED BY
1	CCD	10/26/94	ISSUED								
TENNESSEE VALLEY AUTHORITY											
BROWNS FERRY NUCLEAR PLANT											
UNIT 2											
RESIDUAL HEAT REMOVAL SYSTEM											
SUPPORT LOCATIONS											
DESIGNED BY	REVISED BY	DATE	ISSUED BY	DATE	ISSUED BY	DATE	ISSUED BY	DATE	ISSUED BY	DATE	ISSUED BY
CCD											

Original

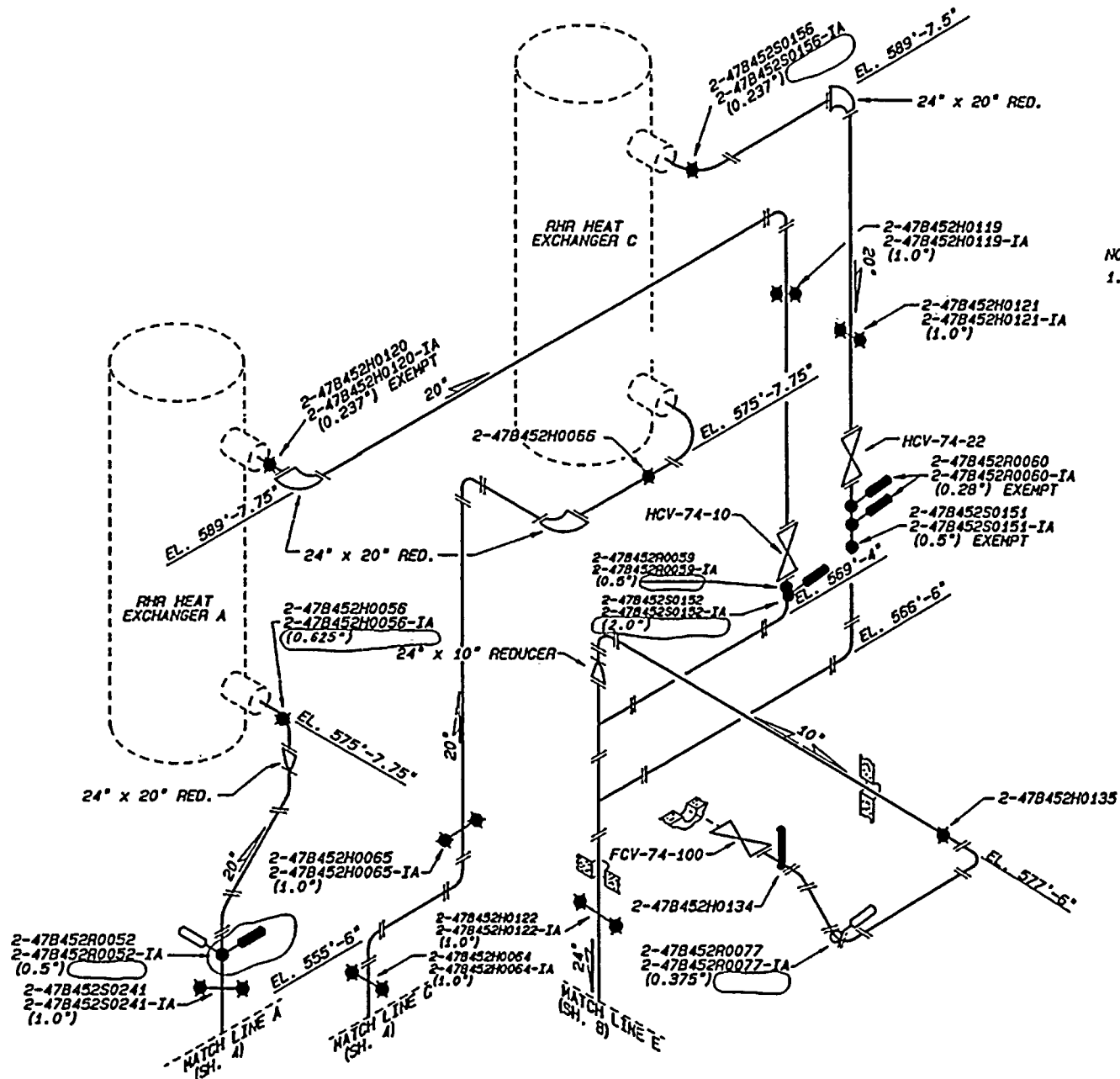


REFERENCE DRAWINGS
 47N452 SERIES
 47N335-7
 MSG-0018-C (SH. 5) WELD MAP
 THIS DRAWING SUPERSEDES MSG-0023-C SH.2
 (SEE NOTE 1)

LEGEND:
 RIGID HANGER
 VARIABLE SUPPORT
 HYDRAULIC SNUBBER
 MECHANICAL SNUBBER
 RIGID STRUT
 ASME CC-2 (EQUIVALENT)

NOTES:





1.) DRAWINGS ISI-0324-C SH. 4-7 MAKE UP
 SUPERCEDED DRAWING MSG-0023-C (SH. 2)



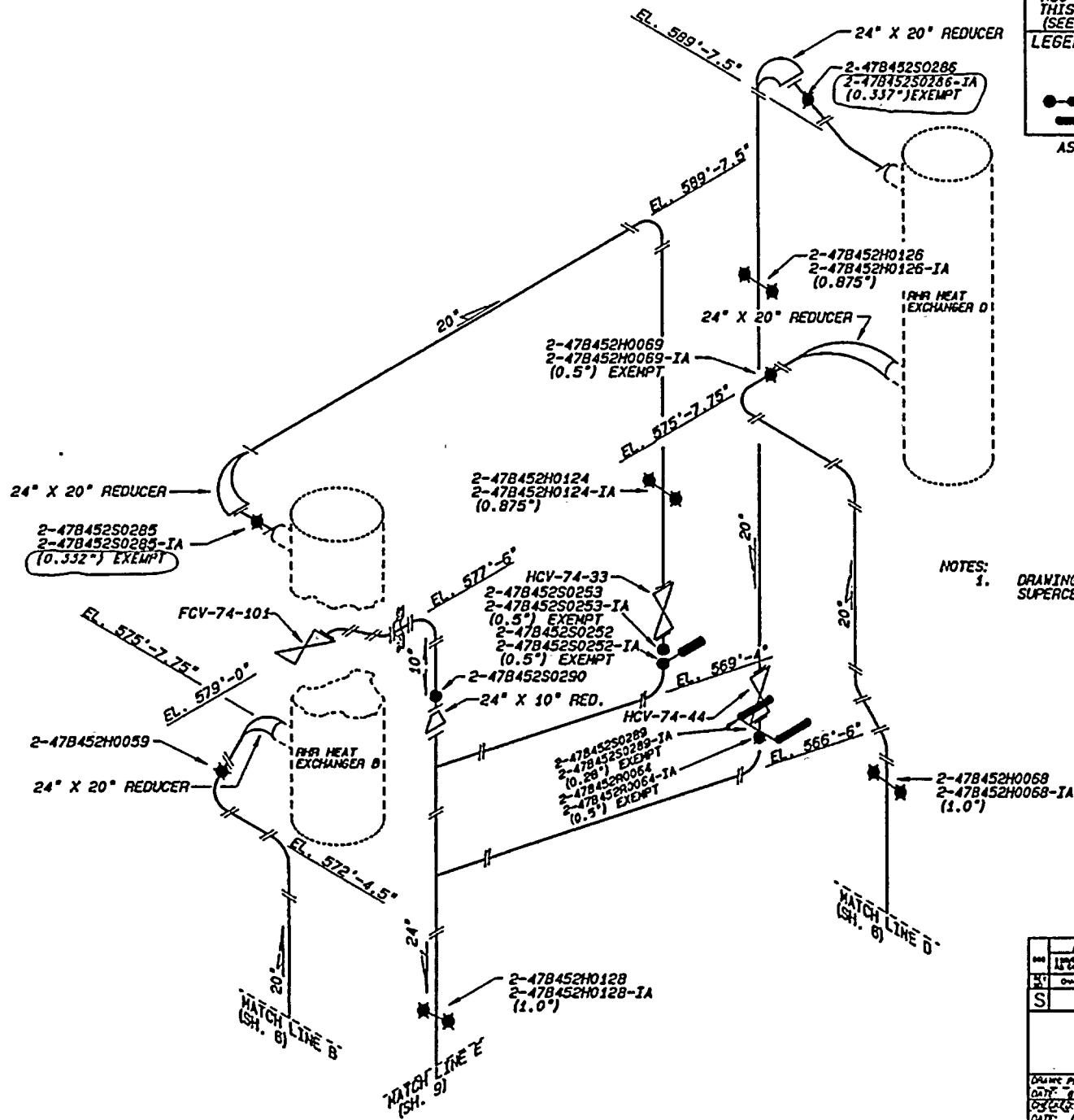
DATE	BY	CHKD	APP'D	DATE	BY	CHKD	APP'D
04/11/80	EDC	GLB		04/11/80	EDC	GLB	
TENSSESEE VALLEY AUTHORITY BROOKS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS							
SCALE	AS SHOWN	DATE	04/11/80	BY	EDC	CHKD	GLB
PROJECT	RESIDUAL HEAT REMOVAL SYSTEM	DRWING NO.	2-151-0324-C	REV.	000	DATE	



REFERENCE DRAWINGS
 47N452 SERIES
 47H335-7
 MSG-0018-C (SH. 7) NEL MAP
 THIS DRAWING SUPERCEDES MSG-0023-C SH. 2
 (SEE NOTE 1)

LEGEND:
 RIGID HANGER
 VARIABLE SUPPORT
 MECHANICAL SNUBBER
 RIGID STRUT

ASME CC-2 (EQUIVALENT)



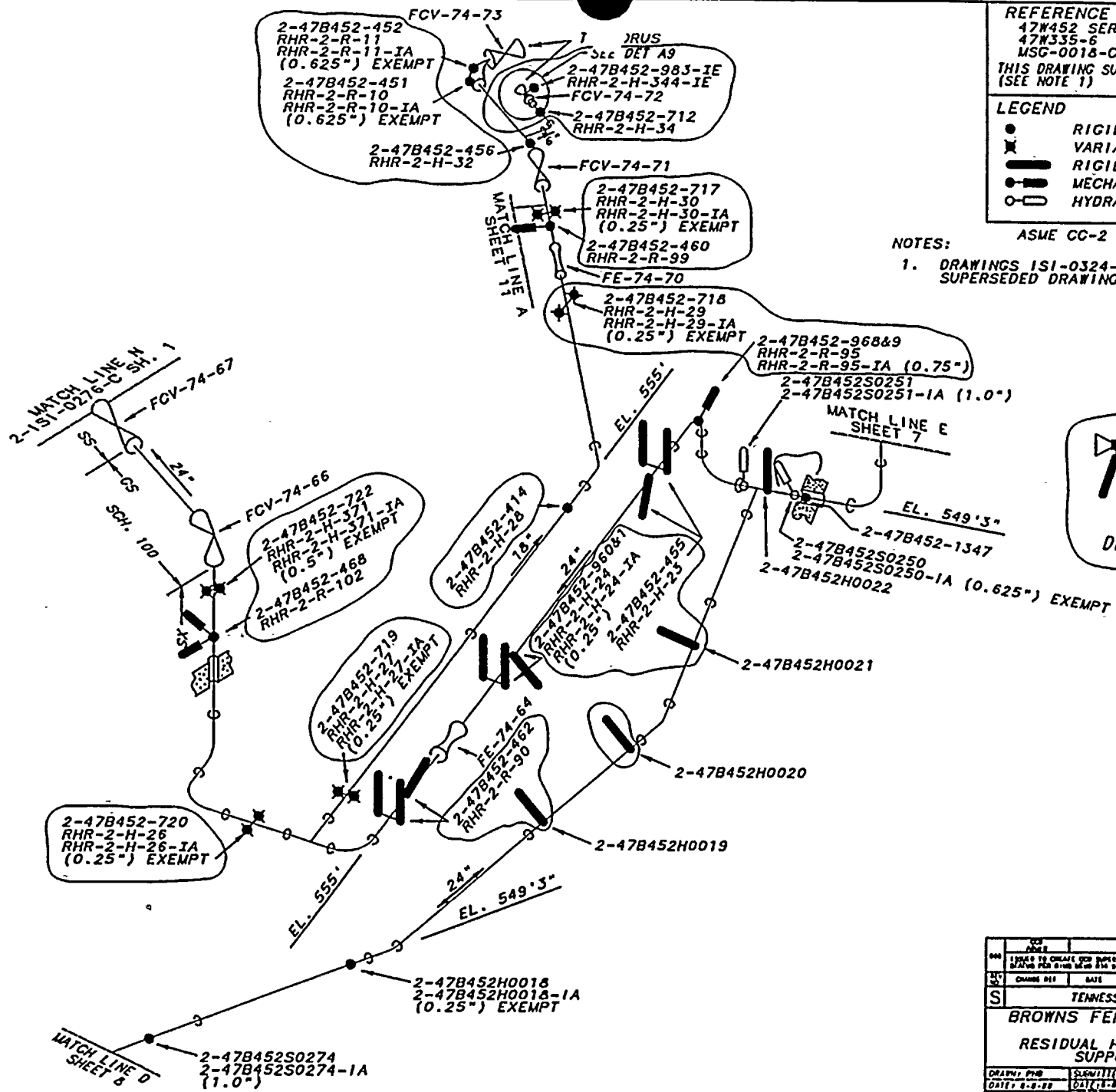
NOTES:
 1.

DRAWINGS ISI-0324-C SH. 4-7 MAKE UP SUPERCEDED DRAWING MSG-0023-C (SH. 2)

REV	DATE	BY	CHK	APP	DESCRIPTION
000					ISSUED FOR CONSTRUCTION
001	08-08	EDC	GLB		EDC
002	08-08	EDC	GLB		EDC

TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 2					
RESIDUAL HEAT REMOVAL SYSTEM					
SUPPORT LOCATIONS					
DRAWING NO.	47N-3110	ISSUED	SCALE	1/2" = 1'-0"	
DATE	8-8-88	DATE	8-8-88	SHEET 1 OF 13	
DESIGNED BY	EDC	CHECKED BY	GLB	DRAWING NO.	
DATE	8-8-88	DATE	8-8-88	2-151-0324-C	

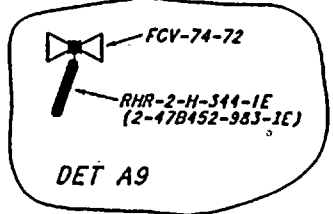




REFERENCE DRAWING
 47W452 SERIES
 47W335-6
 MSG-0018-C (SH. . . WELD MAP
 THIS DRAWING SUPERSEDES MSG-0023-C SH. 3
 (SEE NOTE 1)

- LEGEND
- RIGID HANGER
 - ✱ VARIABLE SUPPORT
 - ▬ RIGID STRUT
 - MECHANICAL SNUBBER
 - HYDRAULIC SNUBBER

NOTES:
 ASME CC-2 (EQUIVALENT)
 1. DRAWINGS ISI-0324-C SH. 8 AND 9 MAKE UP SUPERSEDED DRAWING MSG-0023-C (SH. 3)



REV	DATE	BY	CHKD	APPD	SCALE	DATE	BY	CHKD	APPD
001									
TENSILE TO CHECK FOR COMPLIANCE WITH ASME B31.1 AND TO DETECT AN OVERSTRESS BEARING FOR OTHER BEARING SHALL BE AT 25% STRESS									
002									
TENNESSEE VALLEY AUTHORITY BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL SYSTEM SUPPORT LOCATIONS									
DRWNG: PHB	DESIGNED: JES	APPROVED: CLB	SCALE: NTS						
DATE: 8-8-88	DATE: 8-8-88	DATE: 8-8-88	SCALE: 1/2" = 1'-0"						
CHECKED: JES	DATE: 8-8-88	DATE: 8-8-88	SCALE: 1/2" = 1'-0"						
DATE: 8-8-88	DATE: 8-8-88	DATE: 8-8-88	SCALE: 1/2" = 1'-0"						





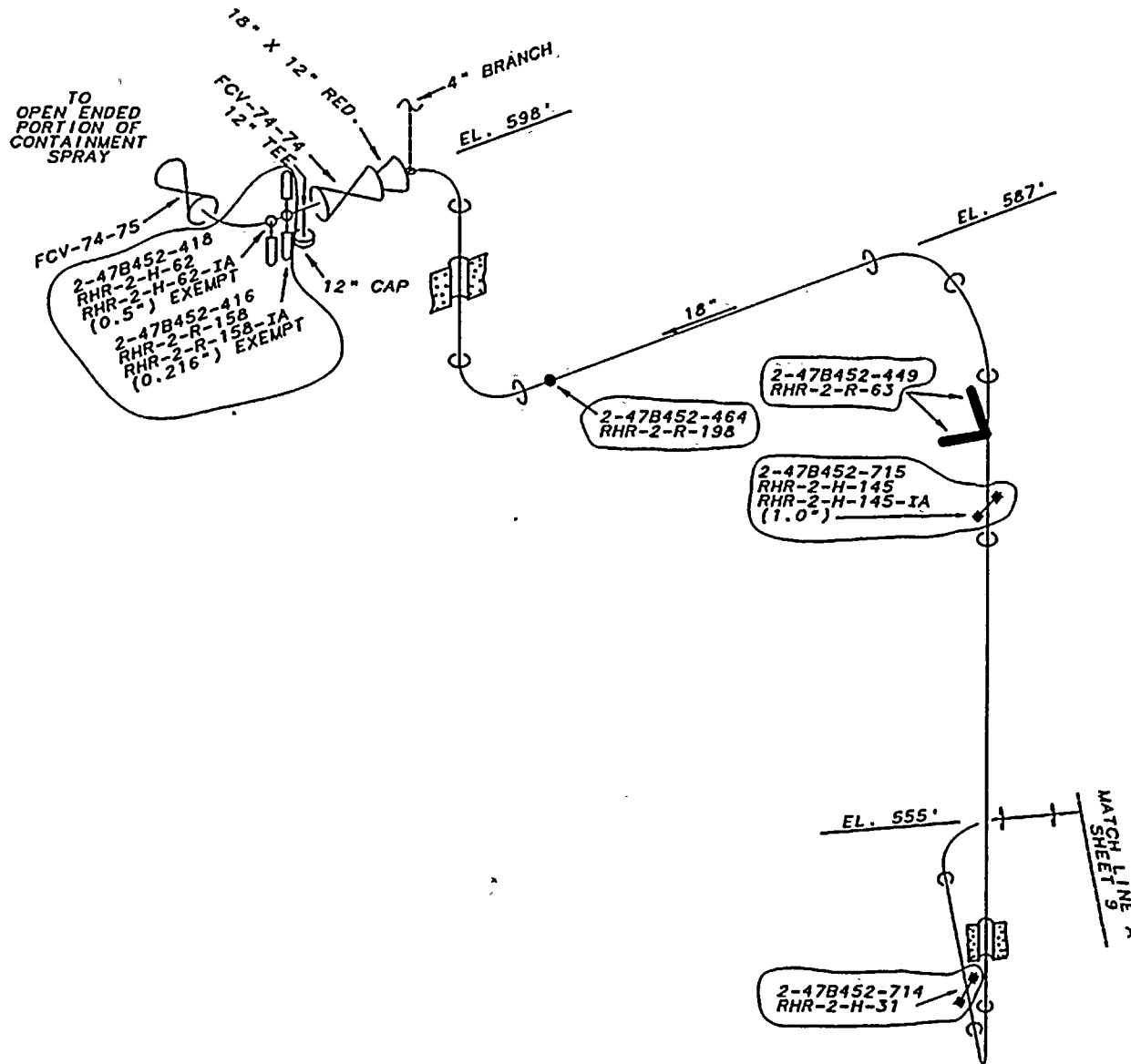
NOTES:

1. DRAWINGS ISI-0324-C SH. 10 AND 11 MAKE UP SUPERSEDED DRAWING MSG-0023-C (SH. 4)

REFERENCE DRAWING
 47B452 SERIES
 47B335-8
 MSG-0018-C (SH. ...) WELD MAP
 THIS DRAWING SUPERSEDES MSG-0023-C SH. 4
 (SEE NOTE 1)

MATERIAL SPECIFICATIONS
 12" X 0.375" NOM. WALL (XS) CS
 18" X 0.500" NOM. WALL (STD) CS

ASME CC-2 (EQUIVALENT)



REV	DATE	BY	CHKD	APPD	SCALE	WTS
001	10-23-77
002
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CALCULATION BRANCH/PROJECT
IDENTIFIER: CD-02073-663012

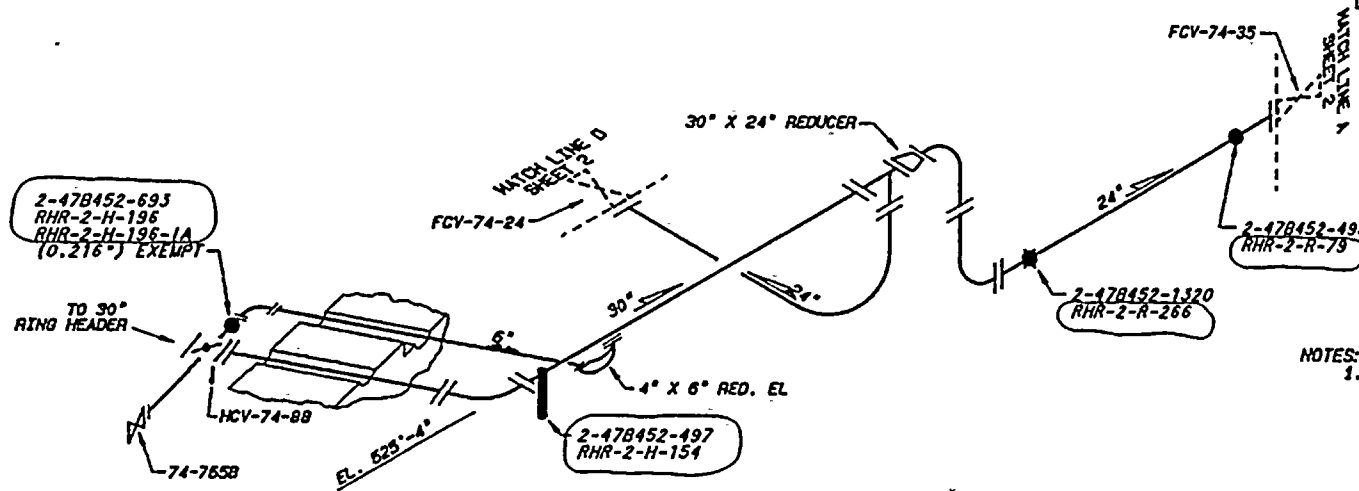
REFERENCE DRAWINGS:

47M403-203
47M403-205
MSG-0018-C (SH. 12) WELD MAP
THIS DRAWING SUPERSEDES MSG-0023-C SH. 1
(SEE NOTE 1)

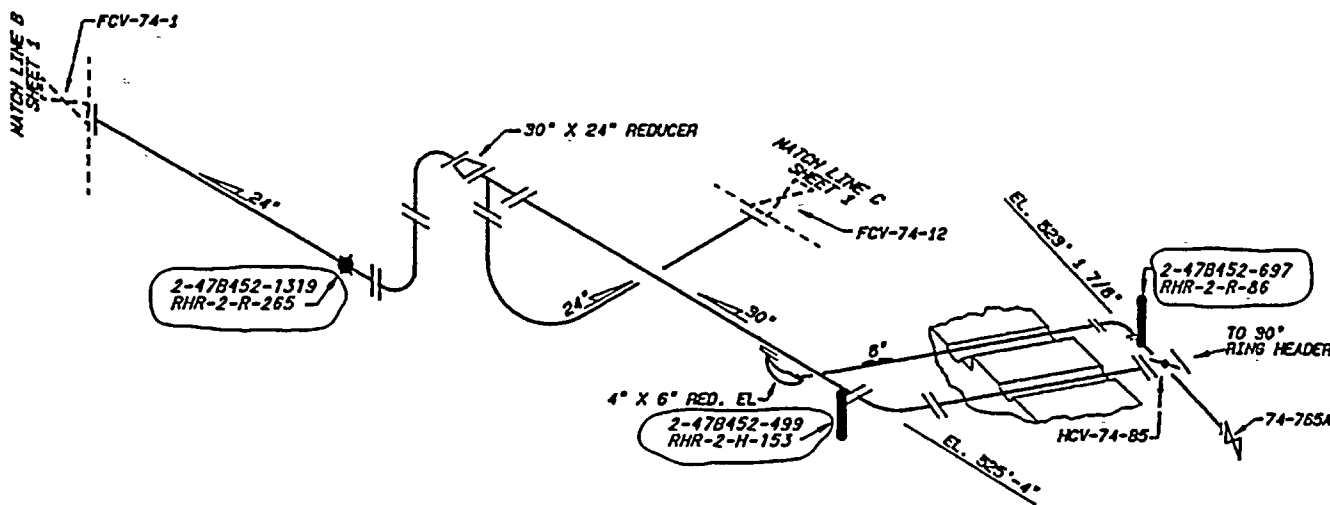
LEGEND:

- RIGID HANGER
- VARIABLE SUPPORT
- RIGID STRUT
- MECHANICAL SNUBBER

ASME CC-2 (EQUIVALENT)



- NOTES:
1.) DRAWINGS ISI-0324-C SH. 1-3, 12
SUPERSEDE DRAWING MSG-0023-C (SH. 1)



REV	DATE	BY	CHKD	APP'D	REASON
1	06-10-88	EDC	GLB		ISSUE FOR CONSTRUCTION

TENNESSEE VALLEY AUTHORITY	
BRONNS FERRY NUCLEAR PLANT	
UNIT #2	
RESIDUAL HEAT REMOVAL SYSTEM	
SUPPORT LOCATIONS	
DATE: 10-1-88	SCALE: NTS
DESIGNER: JEF	QUANTITY NO.:
DATE: 6-10-88	2-TSI-0324-C 1000

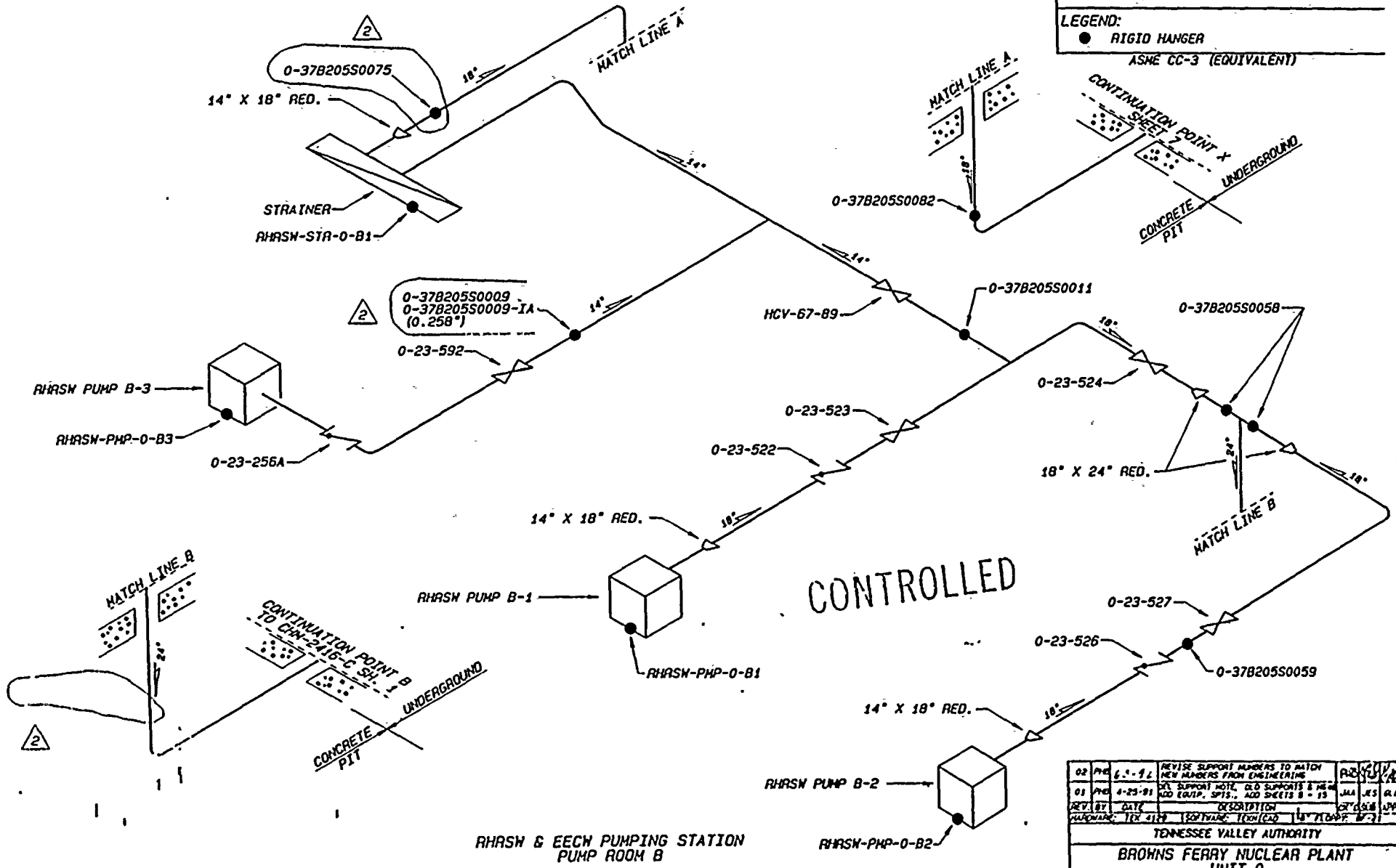


REFERENCE DRAWINGS

- 37B205-417
- 37B205-428
- 0-37B205S0011
- 0-37B205S0058
- 0-37B205S0059
- 0-37B205S0082
- 17B300-11

LEGEND:

- RIGID HANGER
- ASME CC-3 (EQUIVALENT)



RHRSW & EECW PUMPING STATION
PUMP ROOM B

02	REV	6-5-91	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING	RC	CU	AS
01	REV	4-23-91	ADD SUPPORT NOTE, OLD SUPPORTS & NEW ADD EQUIP. SPTS., ADD SHEETS B-13	JMA	JES	RAJ
REV	BY	DATE	DESCRIPTION	DR	CHKD	APP
DRAWING: TERN 4189			SOFTWARE: TERNICAD	LW 7/10/91 P. 2		
TENNESSEE VALLEY AUTHORITY						
BROWNS FERRY NUCLEAR PLANT UNIT 0 EMERGENCY EQUIPMENT COOLING SYSTEM SUPPORT LOCATIONS						
DATE	REV	SUBMITTED	APPROVED	SCALE	SHEET # OF # SHEETS	
04/20/91	JES	04/23/91	JMA	1:1	13 OF 13	
04/20/91	JES	04/23/91	JMA	1:1	13 OF 13	



REFERENCE DRAWINGS

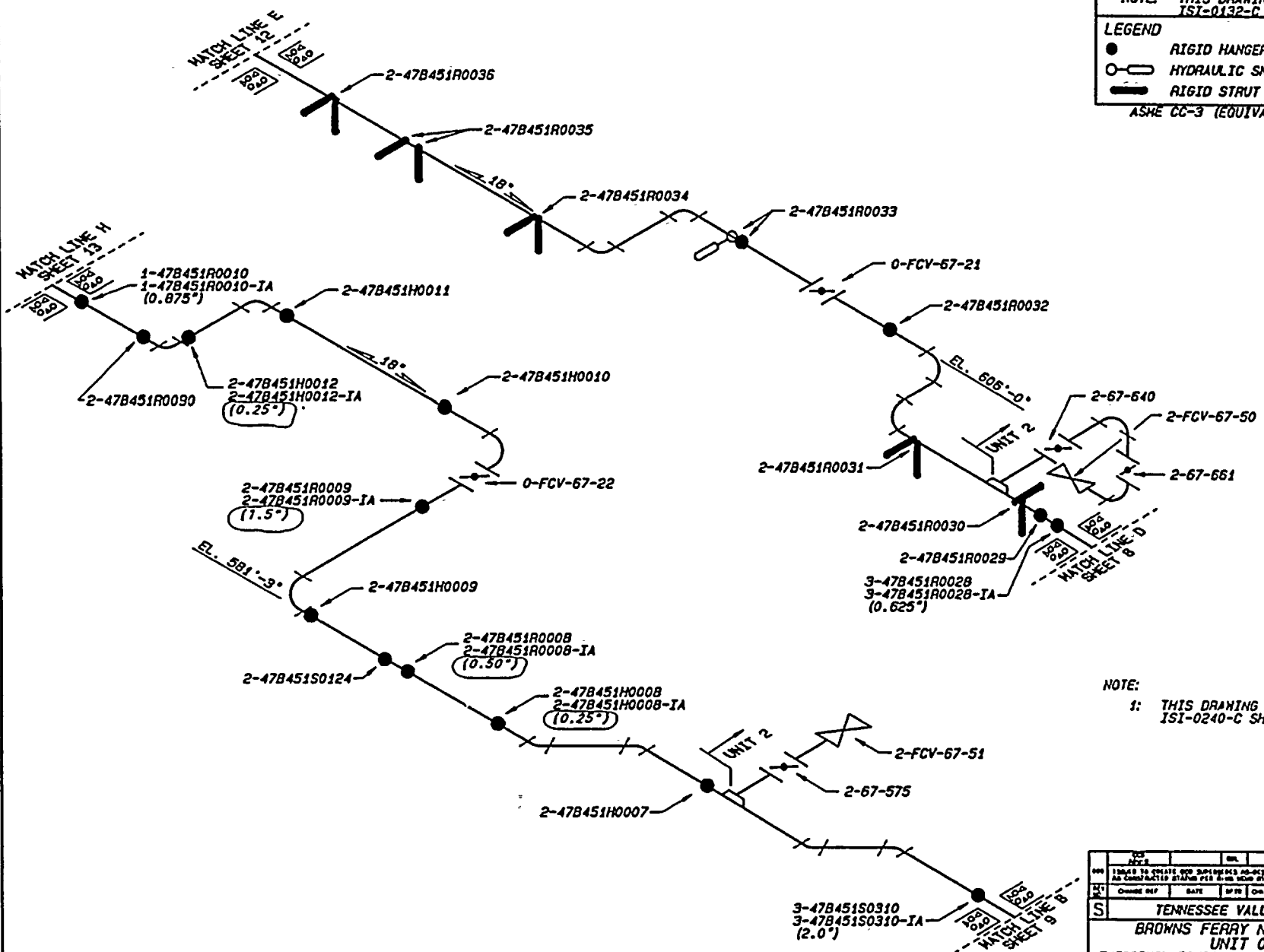
47N451-15

47N451-17

NOTE: THIS DRAWING SUPERCEDES
ISI-0132-C ALL SHEETS

LEGEND

- RIGID HANGER
 - HYDRAULIC SNUBBER
 - RIGID STRUT
- ASME CC-3 (EQUIVALENT)



NOTE:

1: THIS DRAWING SUPERCEDES
ISI-0240-C SH. 1 REV. 4

NO.	DATE	BY	CHKD	APPD	REV	DATE	BY	CHKD	APPD	REV	DATE	BY	CHKD	APPD
1	01-15-91	JAS	KLH											
TENNESSEE VALLEY AUTHORITY														
BROWNS FERRY NUCLEAR PLANT														
UNIT 0 & 2														
EMERGENCY EQUIPMENT COOLING WATER SYSTEM														
SUPPORT LOCATIONS														
SCALE	DATE	BY	CHKD	APPD	REV	DATE	BY	CHKD	APPD	REV	DATE	BY	CHKD	APPD
AS SHOWN	01-15-91	JAS	KLH											
0-151-0368-C 000														

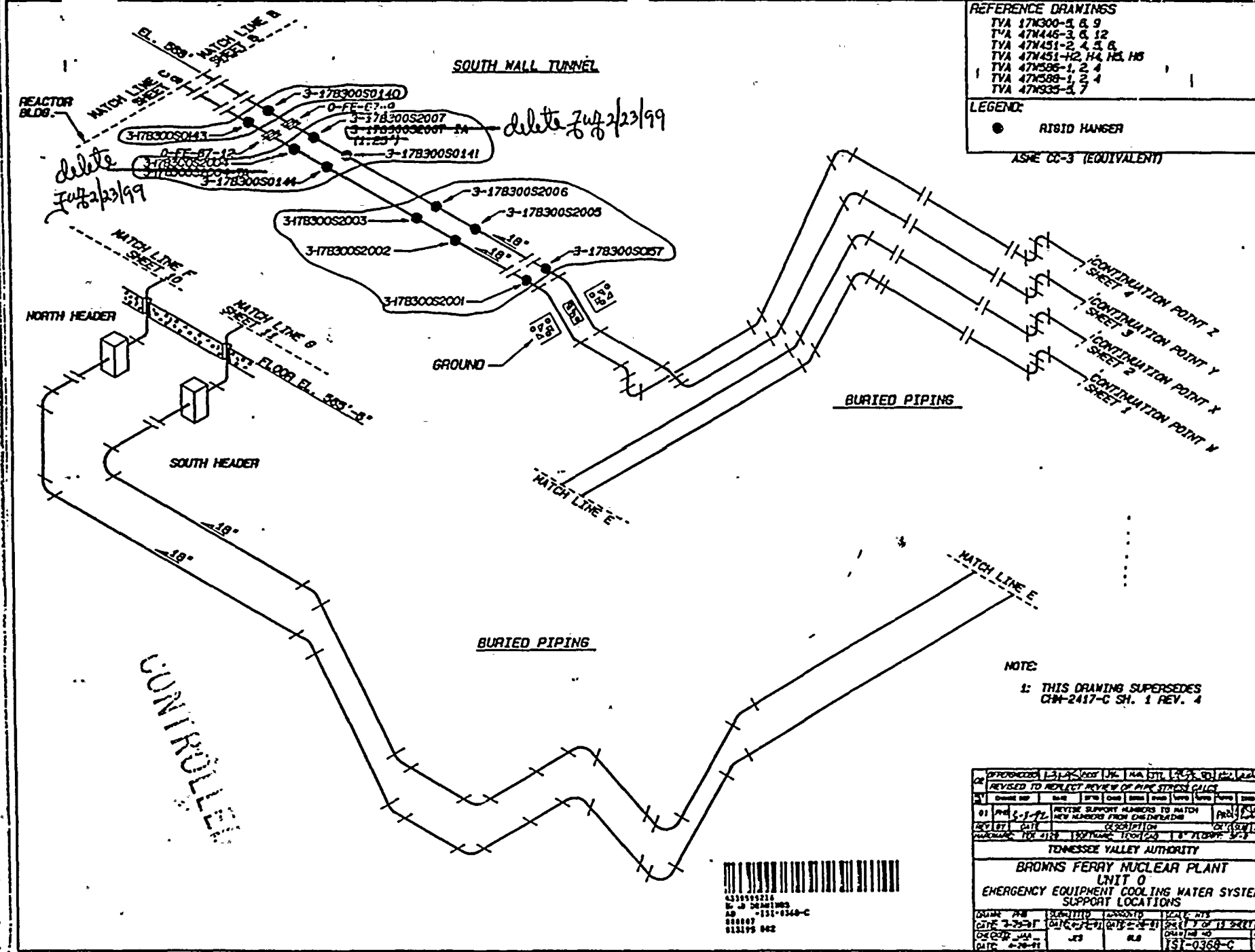


REFERENCE DRAWINGS

TVA 17N300-1, 6, 9
 TVA 47N446-3, 6, 12
 TVA 47N451-2, 4, 5, 6
 TVA 47N451-H2, H4, H5, H8
 TVA 47N586-1, 2, 4
 TVA 47N588-1, 2, 4
 TVA 47N535-1, 7

LEGEND:

● RISID HANGER
 ASME CC-3 (EQUIVALENT)



SOUTH WALL TUNNEL

REACTOR BLDG.

delete July 23/99

delete July 23/99

NORTH HEADER

SOUTH HEADER

GROUND

BURIED PIPING

BURIED PIPING

CONTROLLED

CONTINUATION POINT Z
 SHEET 4
 CONTINUATION POINT Y
 SHEET 3
 CONTINUATION POINT X
 SHEET 2
 CONTINUATION POINT W
 SHEET 1

NOTE:
 1: THIS DRAWING SUPERSEDES
 CHN-2417-C SH. 1 REV. 4

433995214
 E. J. DRAWINGS
 418887
 413175 842

APPROVED	DATE	BY	CHKD	DATE	BY	CHKD	DATE	BY	CHKD
REVISED TO REFLECT REVIEW OF PIPE STRESS ANALYSIS									
01	REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM THE DRAWINGS								
REVISED	DATE	DESCRIPTION	BY	CHKD	DATE	DESCRIPTION	BY	CHKD	DATE
REWORKED	10/18/99	REWORKED 10/18/99	JCS	ALB	10/18/99	10/18/99	JCS	ALB	10/18/99
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT									
UNIT 0									
EMERGENCY EQUIPMENT COOLING WATER SYSTEM									
SUPPORT LOCATIONS									
DATE	BY	DESCRIPTION	DATE	BY	DESCRIPTION	DATE	BY	DESCRIPTION	DATE
DATE 3-29-97	JCS	DATE 3-29-97	JCS	DATE 3-29-97	JCS	DATE 3-29-97	JCS	DATE 3-29-97	JCS
DATE 4-28-97	JCS	DATE 4-28-97	JCS	DATE 4-28-97	JCS	DATE 4-28-97	JCS	DATE 4-28-97	JCS



CALCULATION BRANCH/PROJECT
IDENTIFIER: CD-03067-891725

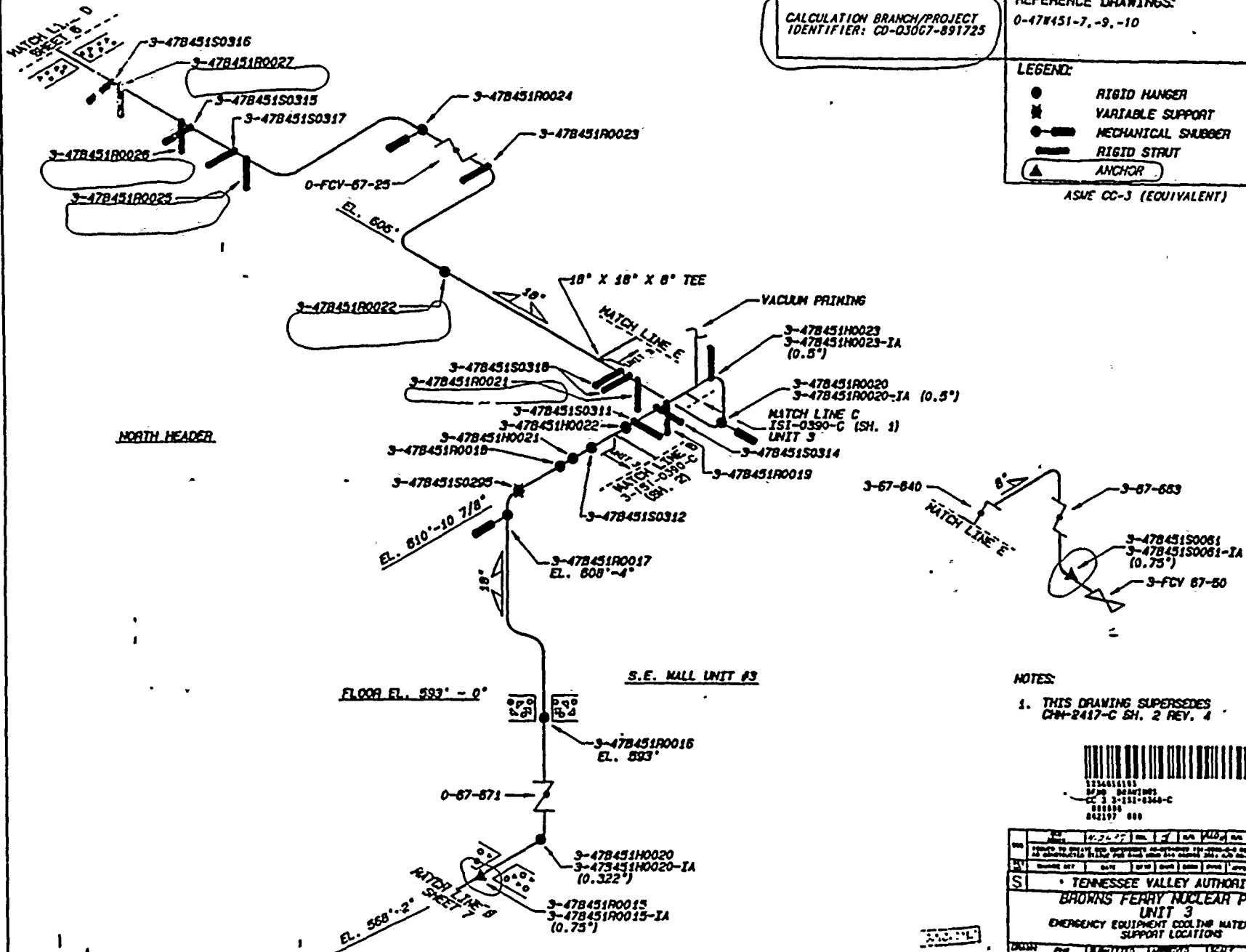
REFERENCE DRAWINGS:

0-478451-7, -9, -10

LEGEND:

- RIGID HANGER
- ⊕ VARIABLE SUPPORT
- MECHANICAL SHABBER
- ▬ RIGID STRUT
- ▲ ANCHOR

ASME CC-3 (EQUIVALENT)



NORTH HEADER

S.E. WALL UNIT #3

FLOOR EL. 593' - 0"

NOTES:

1. THIS DRAWING SUPERSEDES
CHN-2417-C SH. 2 REV. 4



1734816193
BY DR. BEAUMONT
CC 3 3-151-0324-C
011199
042197 000

DATE	BY	CHKD	APP'D	REV	NO.
03/20/88	JCS	EL			
• TENNESSEE VALLEY AUTHORITY BRORNS FERRY NUCLEAR PLANT UNIT 3 EMERGENCY EQUIPMENT COOLING WATER SYSTEM SUPPORT LOCATIONS					
DATE	BY	CHKD	APP'D	REV	NO.
03/20/88	JCS	EL			
3-151-0324-C 000					
CCD					

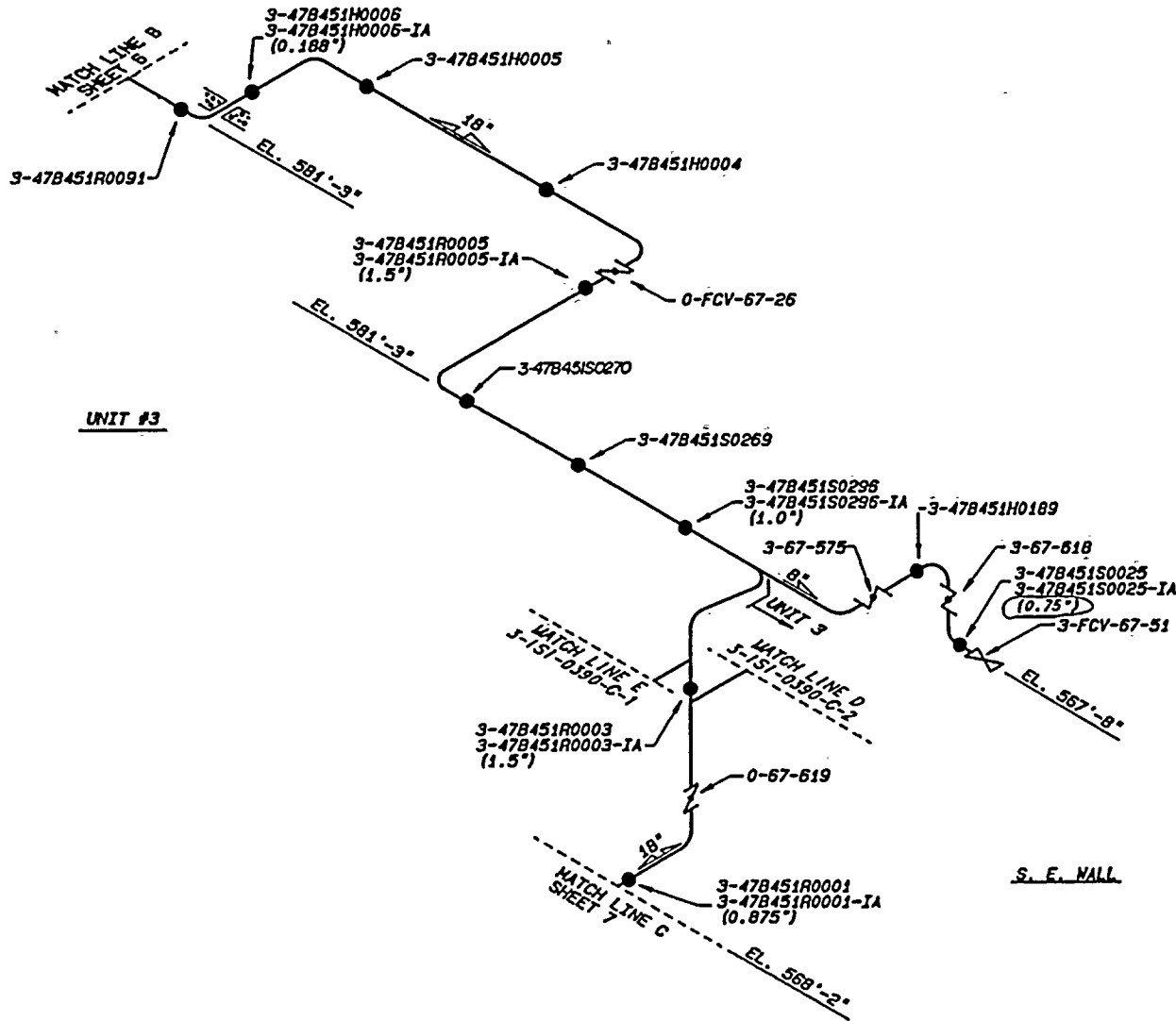


REFERENCE DRAWINGS:
0-478451-8

LEGEND:

● RIGID HANGER

ASME CC-3 (EQUIVALENT)



UNIT #3

UNIT 3

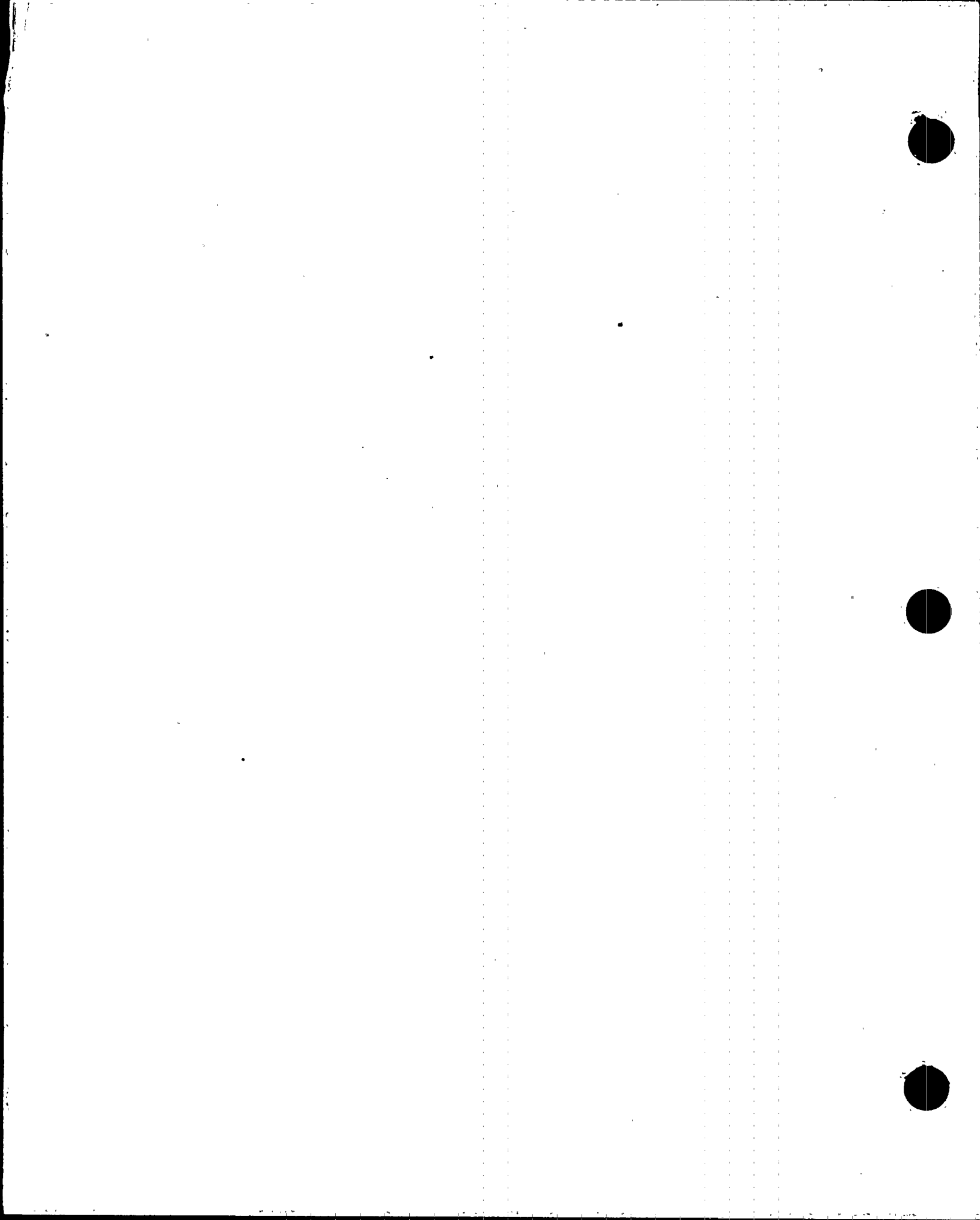
S. E. WALL

NOTES:

1. THIS DRAWING SUPERSEDES CHN-2417-C SH. 3 REV. 4

REV	CHANGE	BY	DATE	APPROVED	DATE
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT					
UNIT 0 & 3					
EMERGENCY EQUIPMENT COOLING WATER SYSTEM					
SUPPORT LOCATIONS					
DRWNG	PHD	DESIGNED	APPROVED	SCALE	NIS
DATE	2-15-81	DATE	2-15-81	SHEET	8 OF 15 SHEETS
DESIGNER	JES	DATE	2-26-81	DRAWING NO.	
DATE	JAN				

CCD	JES	MLB	0-151-0368-C	000
-----	-----	-----	--------------	-----

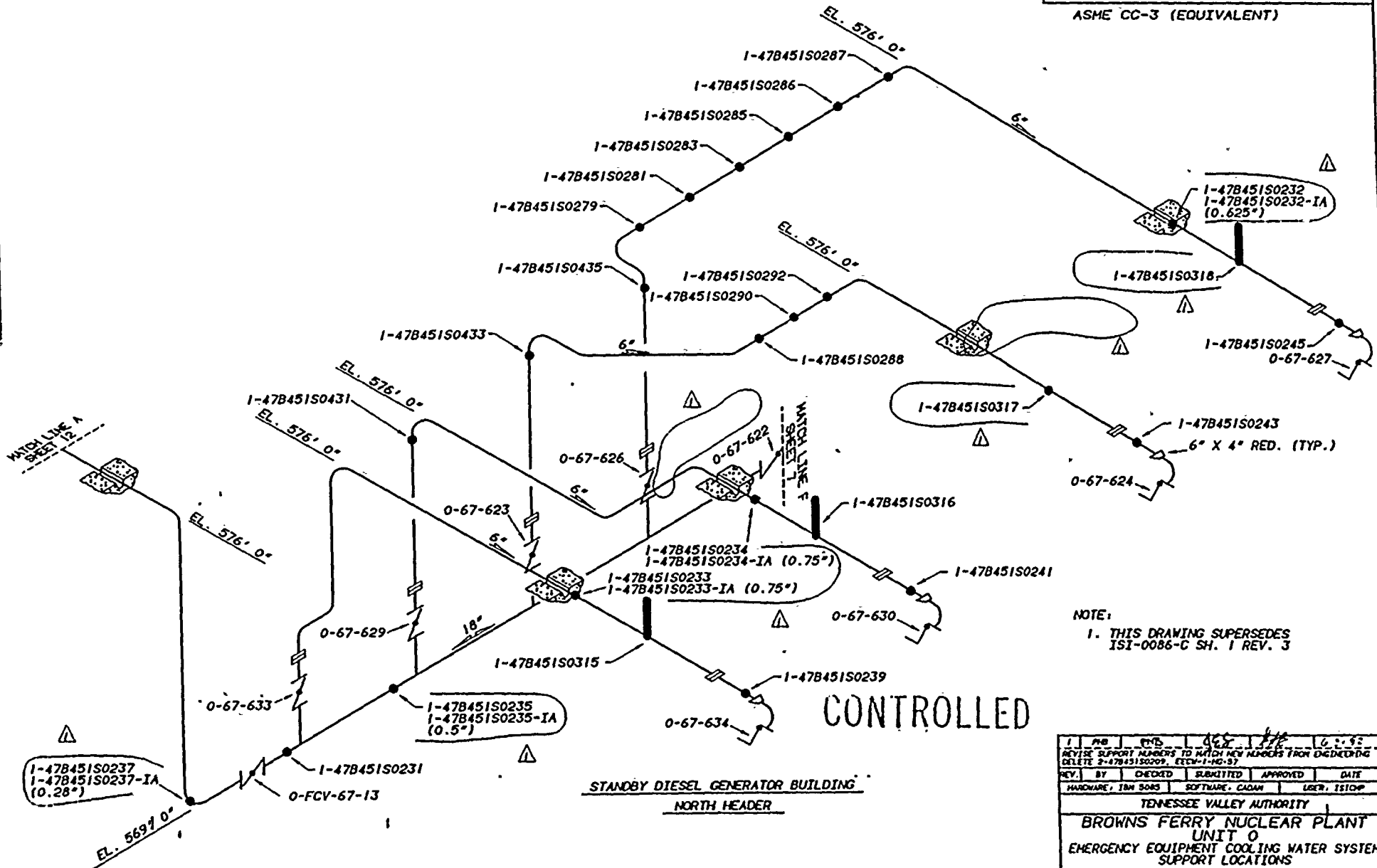


REFERENCE DRAWING
47W586 SERIES

LEGEND

- RIGID HANGER
- ▬ RIGID STRUT

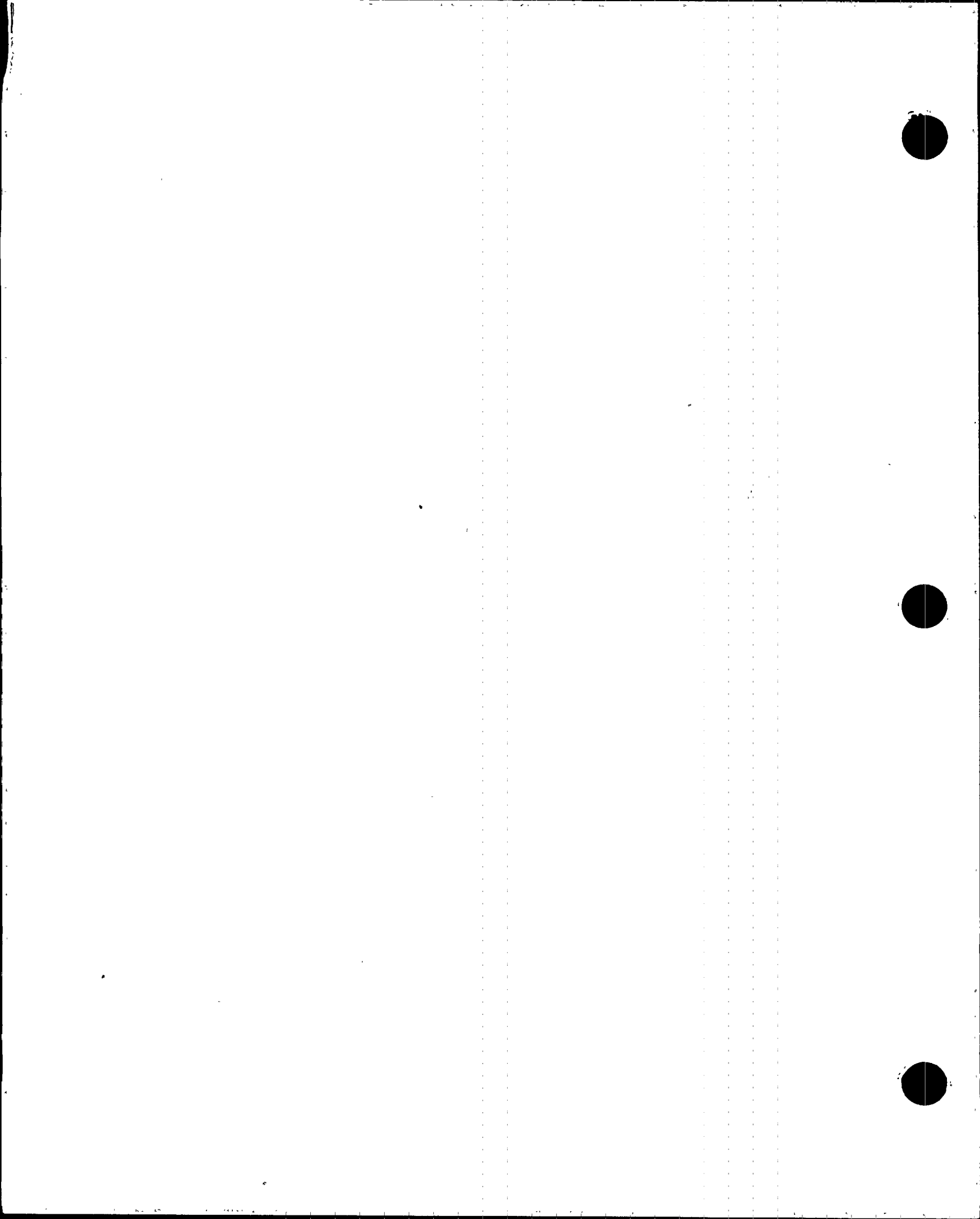
ASME CC-3 (EQUIVALENT)





CONTROLLED

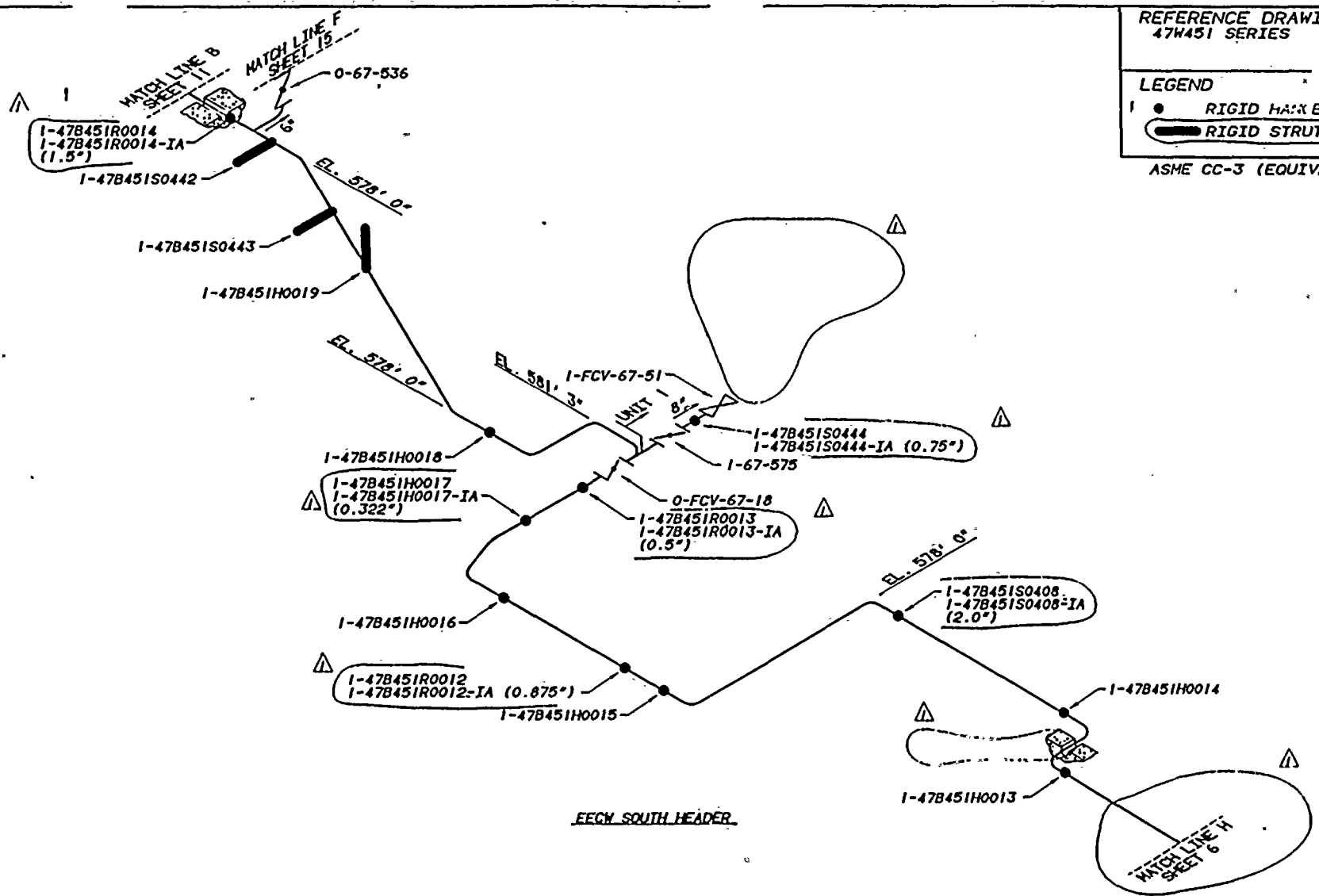
STANDBY DIESEL GENERATOR BUILDING
NORTH HEADER

1	47B	45	0233	0.5	0.5
REVISE SUPPORT NUMBERS TO MATCH REV NUMBERS FROM DECKING DELETE 2-47B45IS0209, ETC-1-40-37					
REV. BY	CHECKED	SUBMITTED	APPROVED	DATE	
HARDWARE, INW 9085	SOFTWARE, CADAM	LGR, ISTOP			
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 0 EMERGENCY EQUIPMENT COOLING WATER SYSTEM SUPPORT LOCATIONS					
DRAWN, PJB	SUBMITTED	APPROVED	SCALE NTS		
DATE, 2-13-91	DATE, 2-18-91	DATE, 2-20-91	SHEET 10 OF 18 PAGES		
DESIGNED, JJA	DATE, 2-18-91	JES	RB	DRAWING NO. ISI-0368-C01	



REFERENCE DRAWING
47H451 SERIES

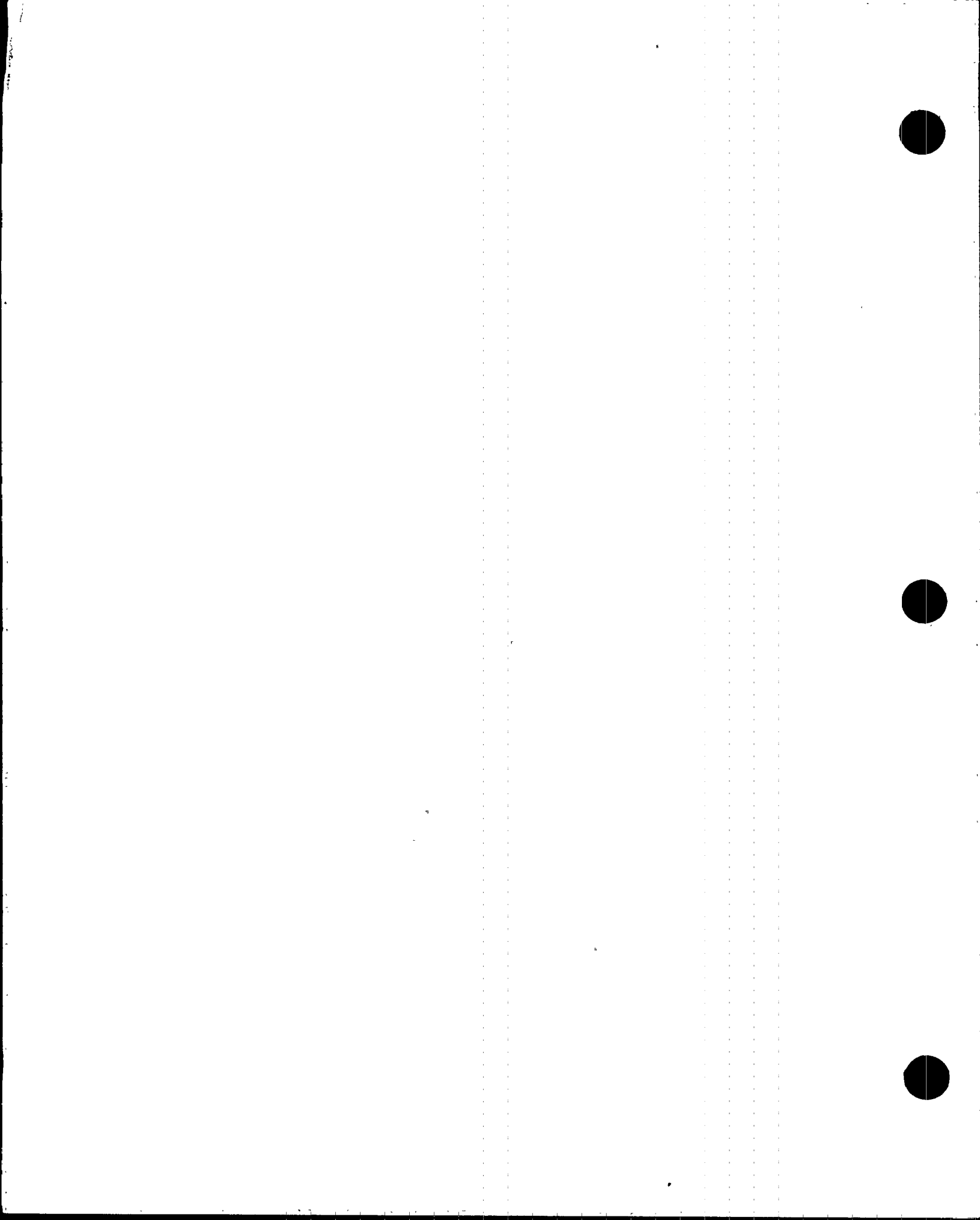
LEGEND
 RIGID HARKER
 RIGID STRUT
 ASME CC-3 (EQUIVALENT)



CONTROLLED

NOTE:
1. THIS DRAWING SUPERSEDES
ISI-0086-C SH. 4 REV. 2

1	PHB	PHB	088	1/16	
REVISE SUPPORT NUMBER TO MATCH NEW NUMBER FROM ENGINEERING CORRECT MATCH LINE H TO SHEET 6, ADD RIGID STRUT TO LEGEND DELETE SUPPORT OUTSIDE BOUNDARY					
REV	BY	CHECKED	SUBMITTED	APPROVED	DATE
HARDWARE	TBM 5043	SOFTWARE	CADAM	USER	ISTOP
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 0 & 1 EMERGENCY EQUIPMENT COOLING WATER SYSTEM SUPPORT LOCATIONS					
DRAWN	PHB	SUBMITTED	APPROVED	SCALE	MIS
DATE	4-15-91	DATE	4-18-91	DATE	4-18-91
CHECKED	JAA	DATE	4-18-91	DRAWING NO	REV
DATE	4-18-91	DATE	4-18-91	DATE	4-18-91
ISI-0368-C01					

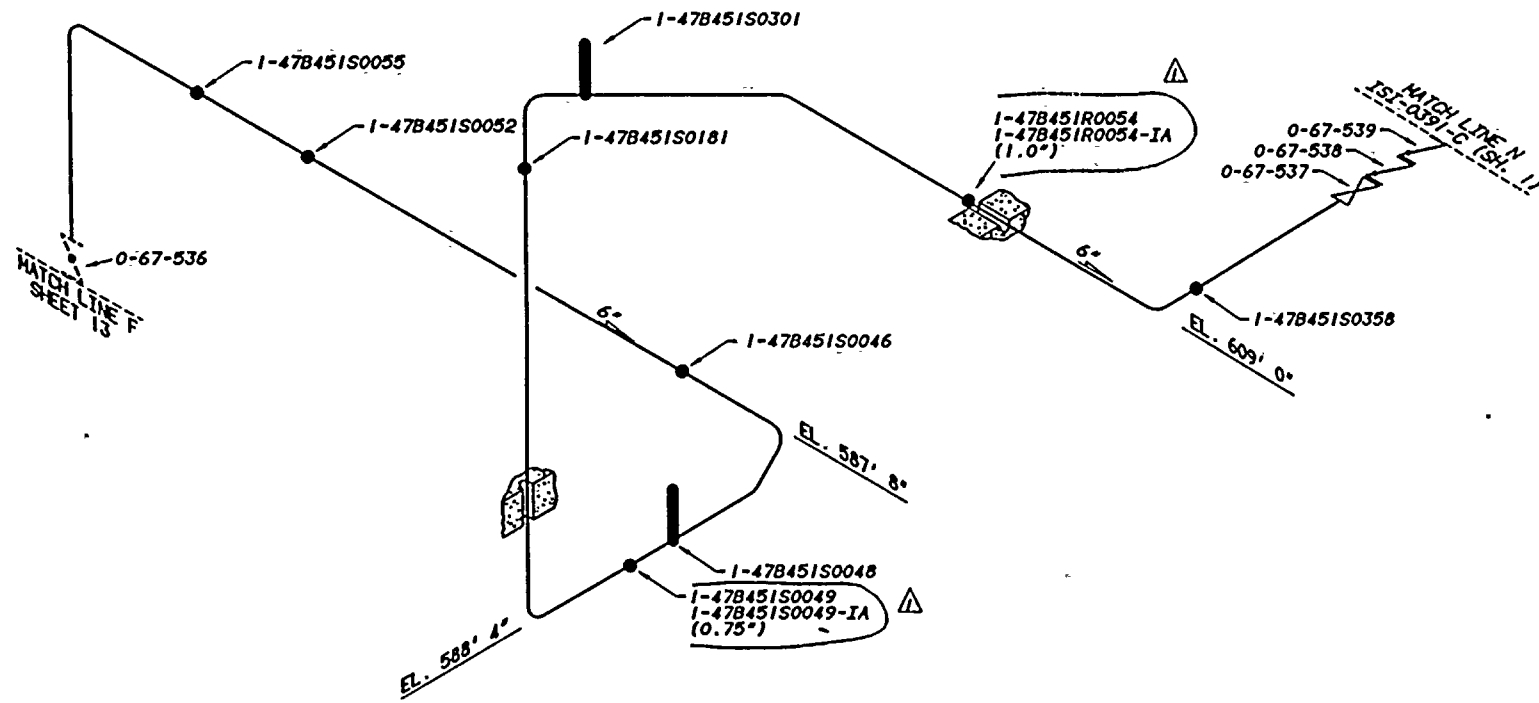


REFERENCE DRAWING
47W451 SERIES
47W935 SERIES

LEGEND

- RIGID HANGER
- ▬ RIGID STRUT

ASME CC-3 (EQUIVALENT)



NOTE:

1. THIS DRAWING SUPERSEDES ISI-0086-C SH. 7 REV. 1

CONTROLLED

1	PHB	PHB	08/27	0/26	2:1:33
REVISE SUPPORT NUMBERS TO MATCH NEW NUMBERS FROM ENGINEERING					
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE
HARDWARE	IBM SOARS	SOFTWARE	CADAM	USER	ISICOP
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 0 EMERGENCY EQUIPMENT COOLING WATER SYSTEM SUPPORT LOCATIONS					
DRAWN	PHB	SUBMITTED	APPROVED	SCALE	NTS
DATE	8-17-97	DATE	8-18-97	SHEET	13 OF 18
DESIGNED	JAA	BY	JES	DRAWING NO.	1000000000

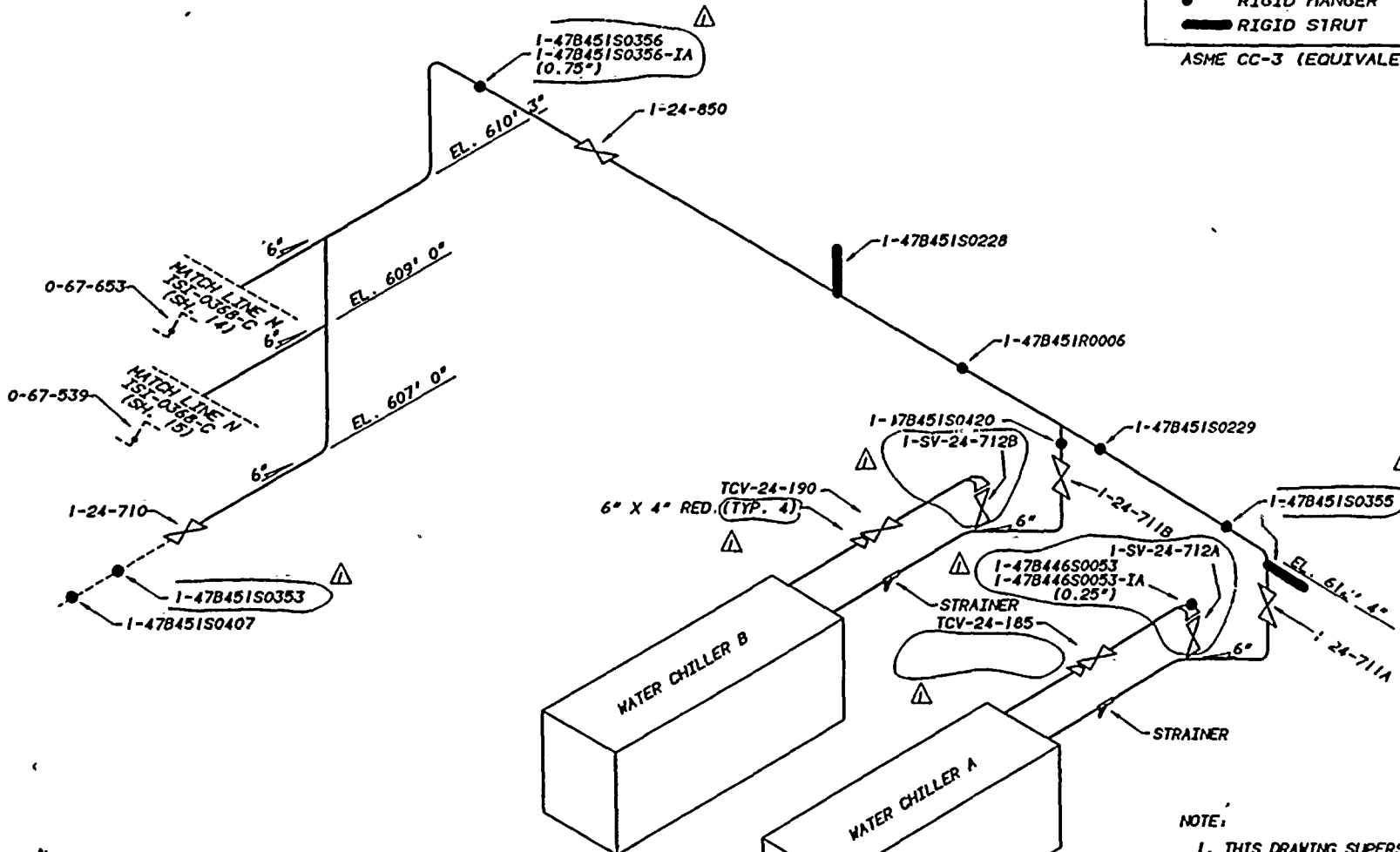


REFERENCE DRAWING 3
 47W935 SERIES
 47W451-202 R1

LEGEND

- RIGID HANGER
- RIGID STRUT

ASME CC-3 (EQUIVALENT)

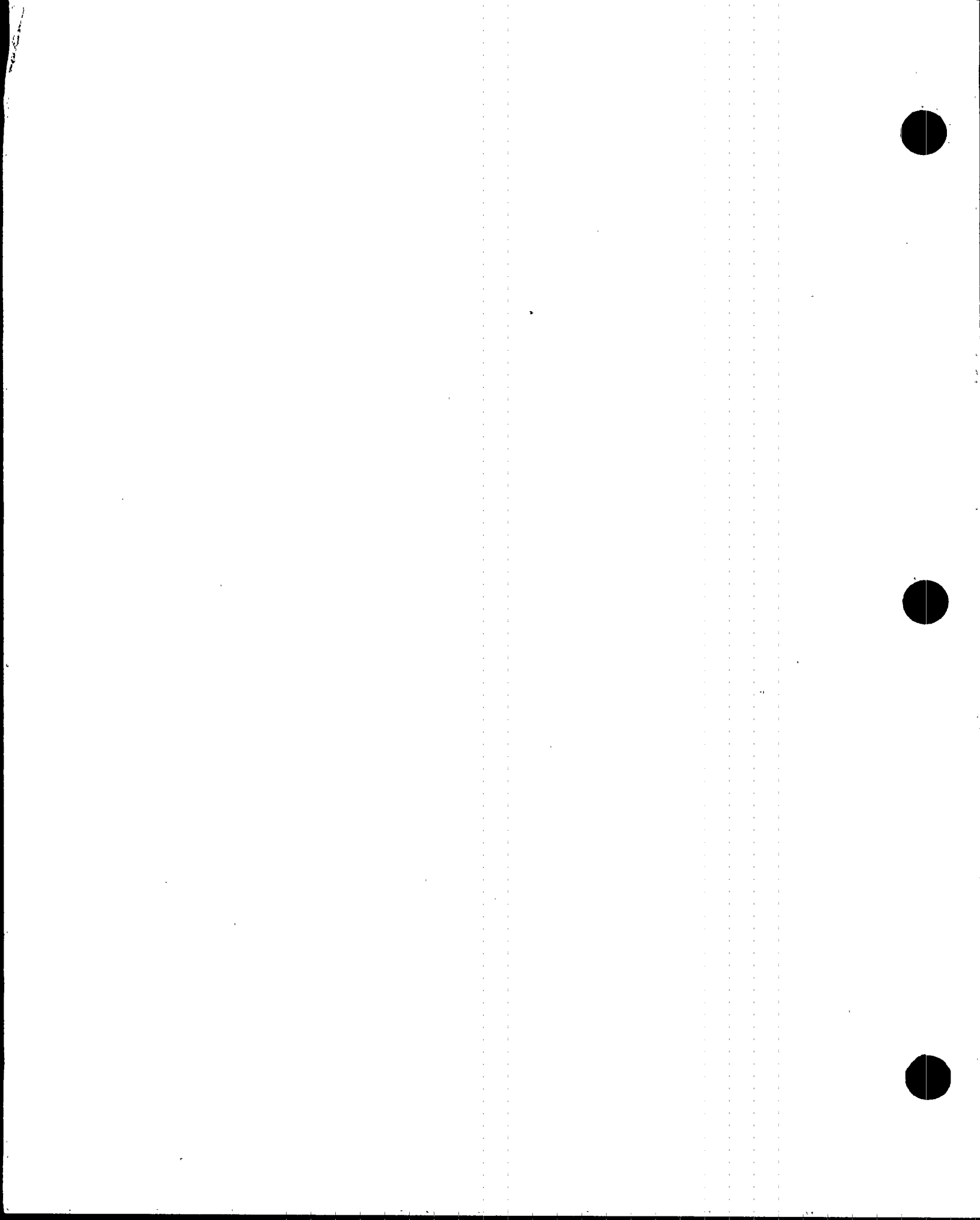


NOTE:

1. THIS DRAWING SUPERSEDES ISI-0086-C SH. 5 REV. 1

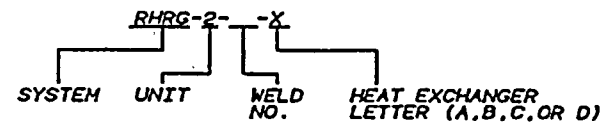
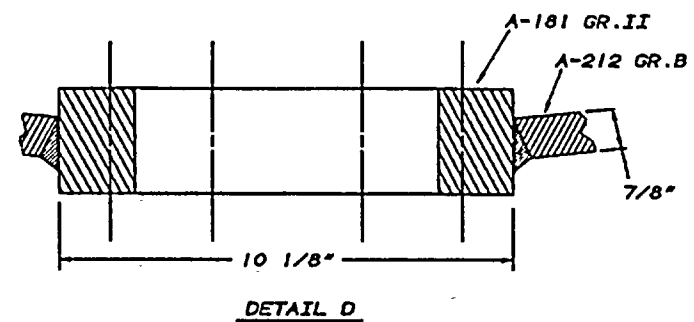
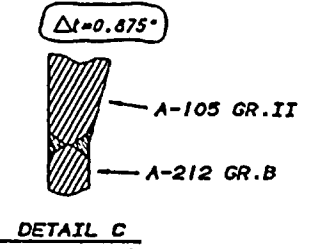
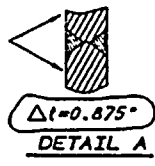
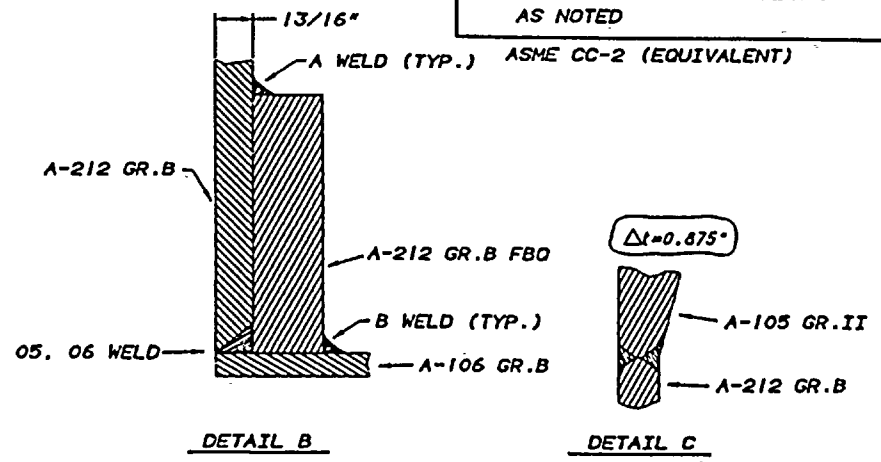
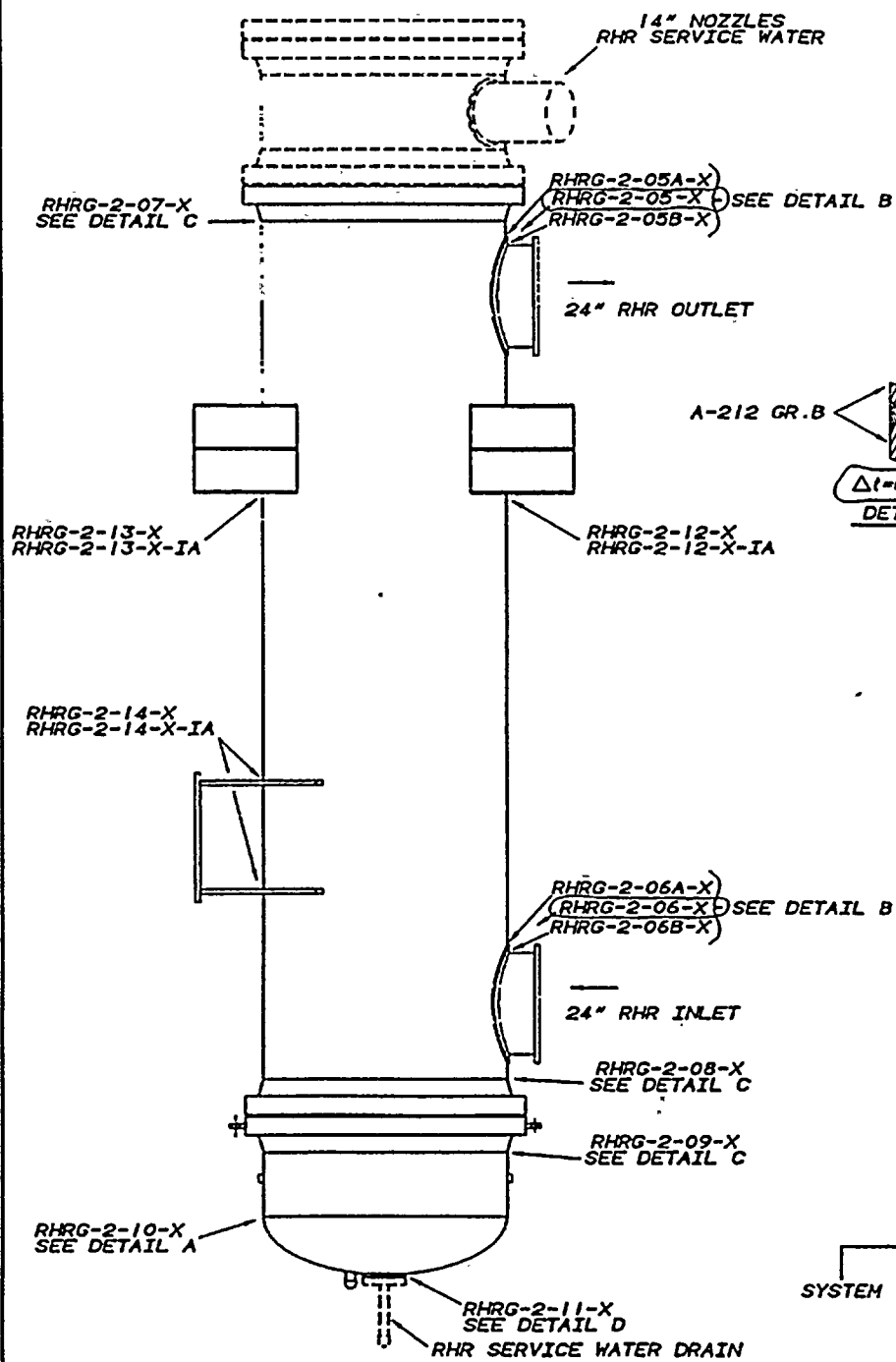
CONTROLLED

1	PHB	JMS	9/5	1/10	1/12
REVISE SUPPORT NUMBER TO MATCH NEW NUMBER FROM ENGINEERING ADD VALUES I-SV-24-712A & B					
REV.	BY	CHECKED	SUBMITTED	APPROVED	DATE
HARDWARE	TOM SORES	SOFTWARE	CADAM	USER	11/10/97
TENNESSEE VALLEY AUTHORITY					
BROWNS FERRY NUCLEAR PLANT UNIT 1 RAW COOLING WATER SYSTEM SUPPORT LOCATIONS					
DRAWN	PHB	SUBMITTED	APPROVED	SCALE	MS
DATE	4-23-91	DATE	4-23-91	DATE	4-23-91
CHECKED	JAA	DATE	4-23-91	DATE	4-23-91
DATE	4-23-91	JMS	CLB	DATE	4-23-91
ISI-0391-C101					



REFERENCE DRAW. 69-BF-165 PERFE. RP. 69-D-160-05 PER. CORP. NOTE: THIS DRAWING SUPERSEDES ISI-0314-B

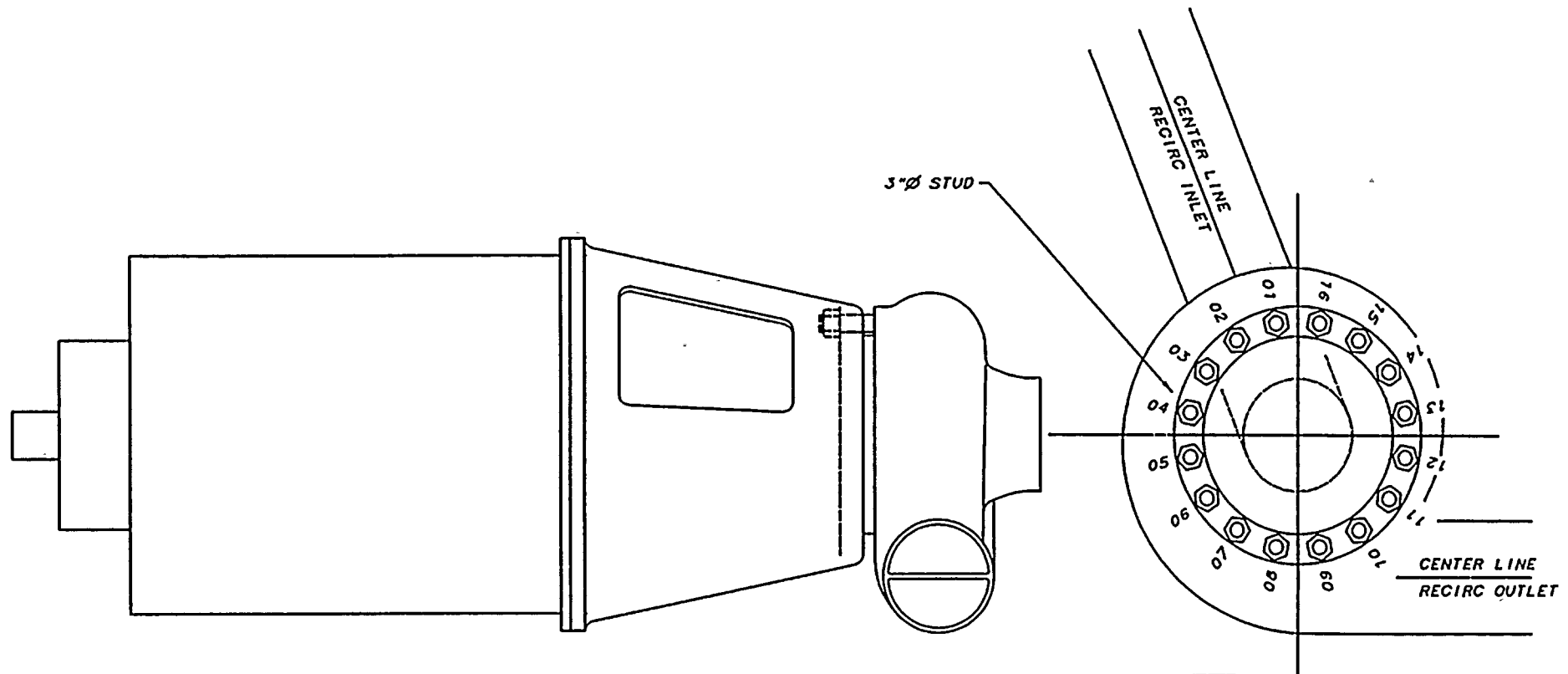
MATERIAL SPECIFICATIONS AS NOTED



ISS	DATE	BY	CHKD	DATE	BY	CHKD	DATE	BY	CHKD
ISSUED TO CREATE CCD SUPPLEMENT AS-DRAWN ISI-0406-C-1 BY RPD TO REFLECT AS CONTRACTED STATUS PER RHR WELD A14 CHANGE 261, A15 RE-AT									
REV	CHANGE NO.	DATE	BY	CHKD	DATE	BY	CHKD	DATE	BY
S									
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT UNIT 2 RESIDUAL HEAT REMOVAL HEAT EXCHANGER WELD LOCATIONS									
DRAWN: PFB	DATE: 3-13-92	SCALE: N/A	ISSUED BY: JES						
CHECKED: RPS	APPROVED: CLB	SHEET 01 OF 01							
SUBMITTED: JES		2-ISI-0406-C		000					
CCD									



REFERENCE DRAWINGS
 2F-1177 BYRON JACKSON , BORG-WARNER
 1E-3429 BYRON JACKSON D... BORG-WARNER
 153F754 GENERAL ELECTRIC
 ASME CC-1 (EQUIVALENT)



NOTE:
 1. MATERIAL ASTM A510 GR B23.

PUMP A
NUMBERS (01-16) ARE PREFIXED BY:

PUMP A.....PMP-A-STUD-2-
 PUMP A.....PMP-A-NUT-2-
 PUMP A.....PMP-A-WASH-2-
 PUMP A.....PMP-A-FLG-2-

PUMP B
NUMBERS (01-16) ARE PREFIXED BY:

PUMP B.....PMP-B-STUD-2-
 PUMP B.....PMP-B-NUT-2-
 PUMP B.....PMP-B-WASH-2-
 PUMP B.....PMP-B-FLG-2-

PUMP A
PUMP INTERIOR IDENTIFIERS:
 PUMP A.....PMP-2A-INTERIOR

PUMP B
PUMP INTERIOR IDENTIFIERS:
 PUMP B.....PMP-2B-INTERIOR

REV	DATE	BY	CHKD	APPD	DATE	BY	CHKD	APPD	DATE
001									
I HAVE TO DATE FOR APPROVAL AS REQUIRED 151-4447-2-1 BY AND TO REPORT AS CONSTRUCTION STAFF FOR AND WITH THE STATION AND TO									
REV	CHANGE REF	DATE	BY	CHKD	APPD	DATE	BY	CHKD	APPD
S									
TENNESSEE VALLEY AUTHORITY									
BROWNS FERRY NUCLEAR PLANT UNIT 2									
RECIRCULATION PUMP BOLTING LOCATIONS & PUMP INTERIORS									
DRWN: PNB	SUBMITTED	APPROVED	SCALE: NTS						
DATE: 3-12-72	DATE: _____	DATE: _____	SHEET 01 OF 01 SHEETS						
CHECKED: BPC	DATE: _____	DATE: _____	DATE: _____						
DATE: _____	DATE: _____	DATE: _____	DATE: _____						
2-151-0407-C-000									
CCD									



REFERENCE DRAWINGS
 B&W 122876
 B&W 122877
 NOTE: THIS DRAWING SUPERSEDES
 ISI-0294-A SH 1

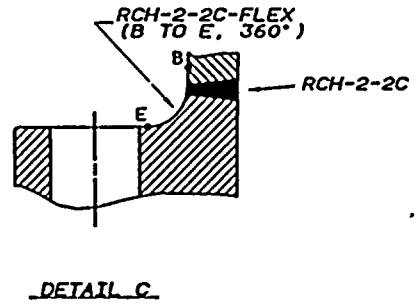
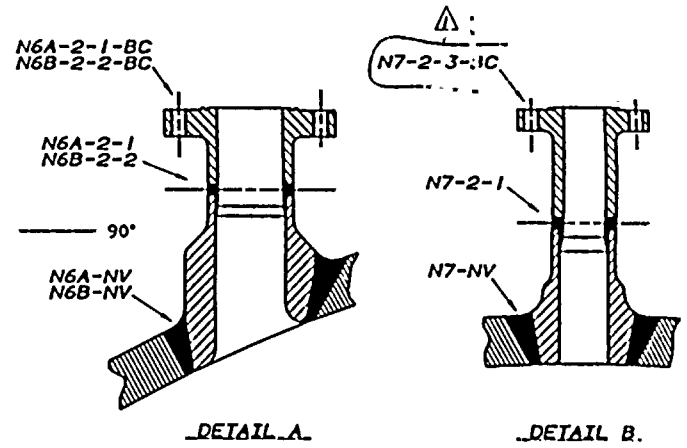
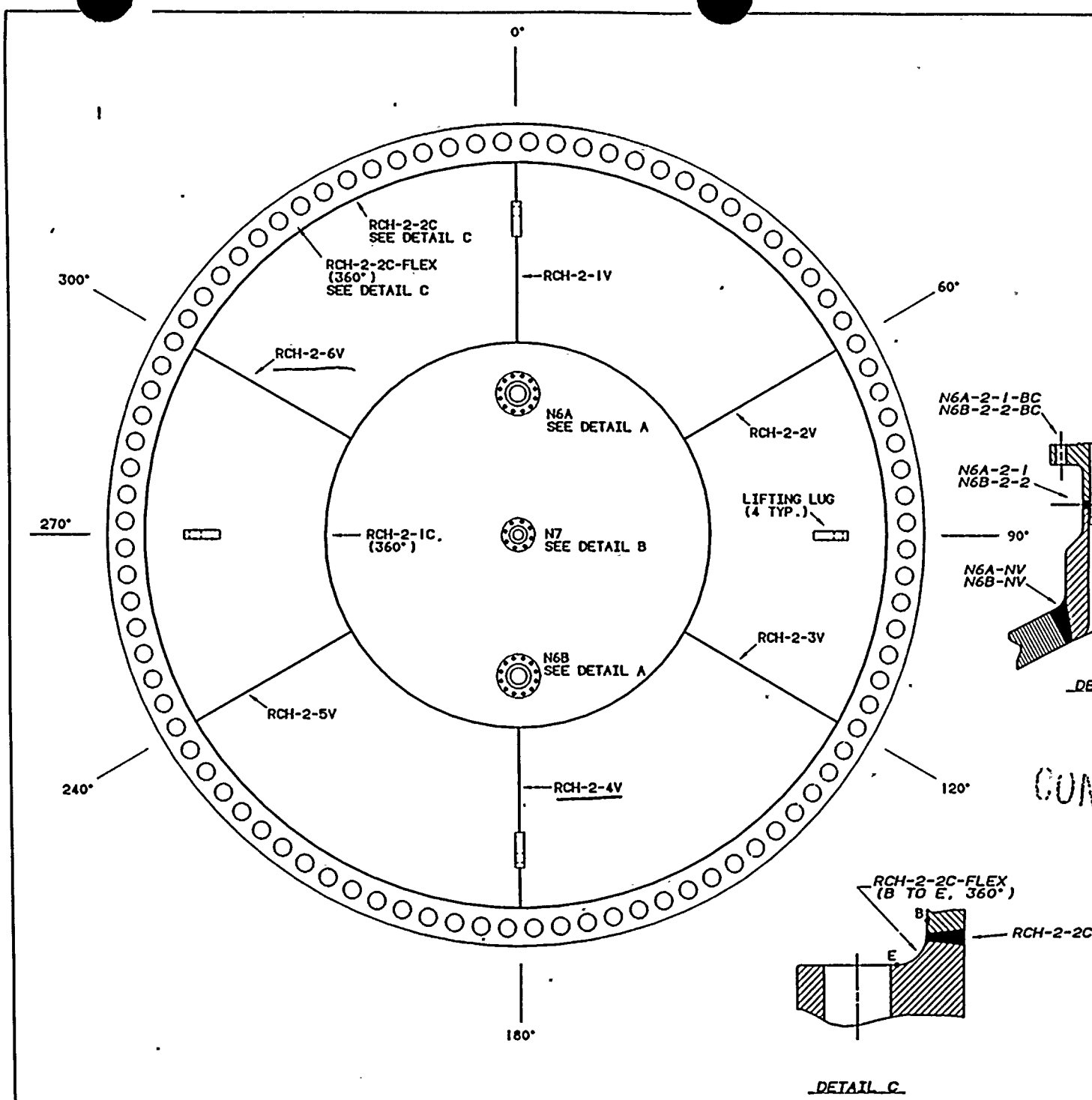
MATERIAL SPECIFICATIONS

CLOSURE HEAD DOME/SEGMENTS
 CS MN/MO

N-7
 NOZZLE - A508 CL.2 (MN-MO)
 LONG WELD NECK - SA-105 GR.II CS

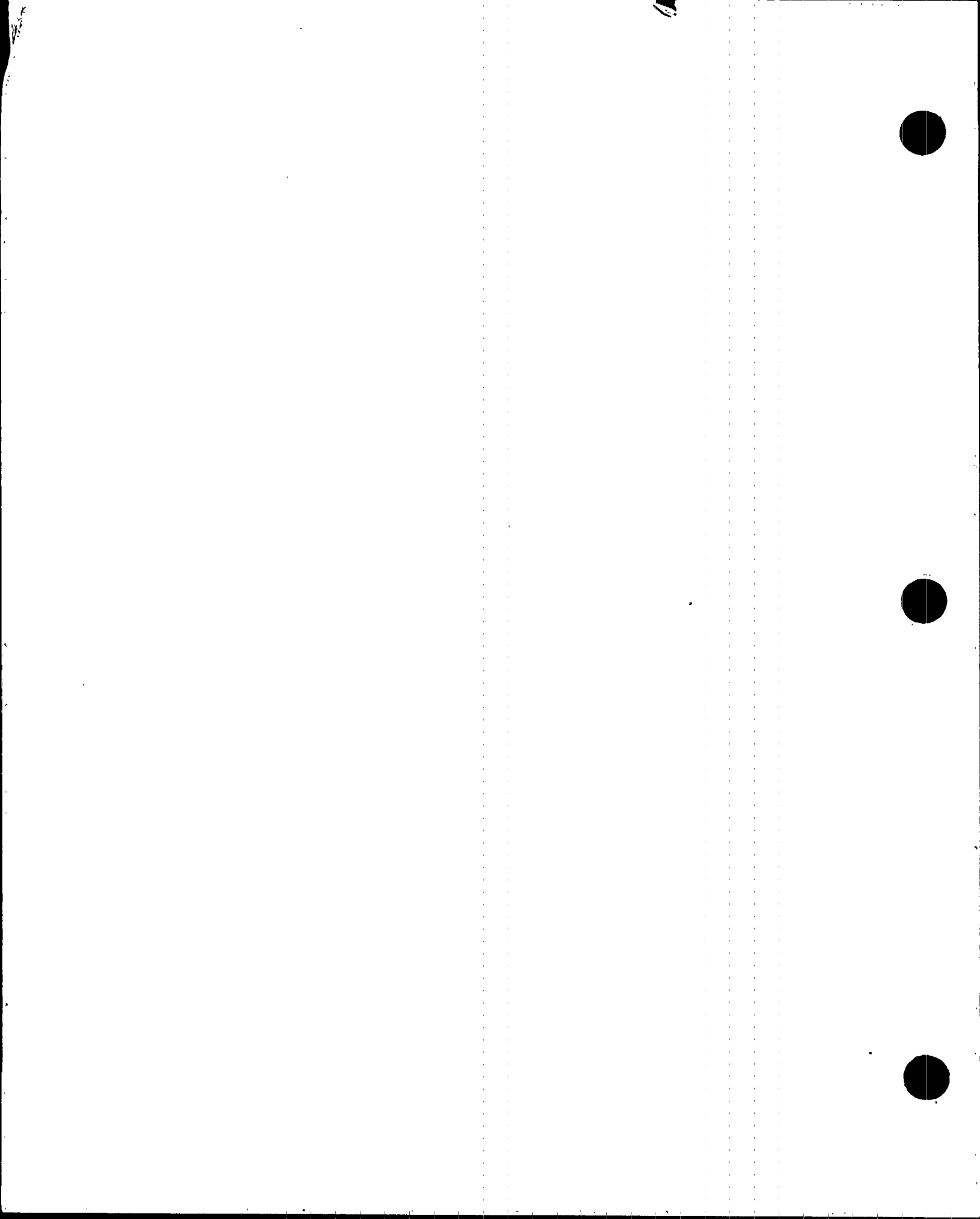
N6A, N6B
 NOZZLE - A508 CL.2 (MN-MO)
 LONG WELD NECK - SA-105 GR.II CS

ASME CC-1 (EQUIVALENT)



CONTROLLED

1	PJB	12/22/92	12/22/92	12/22/92	12/22/92	12/22/92	12/22/92
REVISE NOZZLE N7 BOLTED CONNECTION ID							
REV.	BY	CHECKED	DESIGNED	APPROVED	DATE	DATE	DATE
HARDWARE	IBM 5085	SOFTV	E. CADAN	USER	ISTOP		
TENNESSEE VALLEY AUTHORITY							
BROWNS FERRY NUCLEAR PLANT UNIT 2 CLOSURE HEAD ASSEMBLY WELD LOCATIONS							
DRAWN	PJB	SUBMITTED	APPROVED	SCALE	NIS		
DATE	3-13-92	DATE	3-13-92	DATE	3-13-92	SHEET	01 OF 01
CHECKED	JPC	DATE		DRAWING NO.			
DATE		JES	GLB	ISI-0408-C01			



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

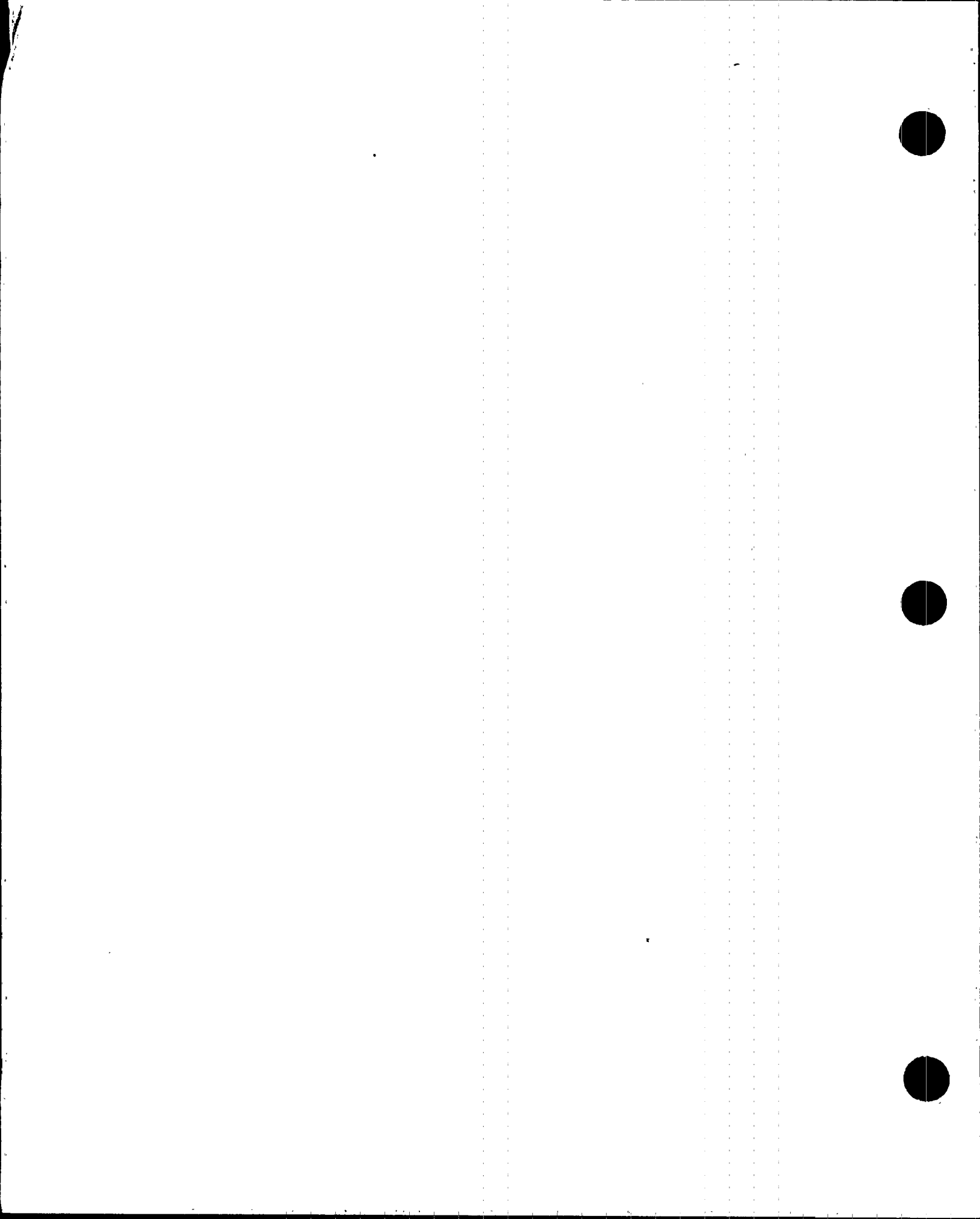
UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

APPENDIX VII

SUMMARY OF INDICATIONS
ADDITIONAL SAMPLES.



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
 OFFICE OF NUCLEAR POWER PO BOX 2000
 1101 MARKET STREET DECATUR, ALABAMA 35609-2000
 CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

SUMMARY OF INDICATIONS

Indications detected during the performance of examinations for Browns Ferry Nuclear Plant unit 2 cycle 10 were evaluated in accordance with approved written procedures. Generally, examination results yielded either No Recordable Indications (NRI) or Recordable Indications.

Recordable Indications were evaluated to determine their origin. Indications determined to be of a geometric, metallurgical, or similar origin were dispositioned as non-relevant. Indications determined to be of a non-geometric, non-metallurgical, or similar origin were dispositioned as relevant and required additional measures such as further evaluation in accordance with ASME Section XI acceptance standards, engineering analysis, repair, or replacement.

The following list is a summary of unacceptable indications detected and corrective measures taken during the unit 2 cycle 10 outage.

NOI NO.	CODE CAT.	COMPONENT IDENTIFIER	INDICATION DESCRIPTION	RESOLUTION	ADDITIONAL SAMPLES
U2C10-001	F-A	2-47B450H0042	SETTING OUT OF RANGE & BROKEN TACK WELD	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (setting acceptable per calcs* CD-Q2023-891351 Rev. 5 & CD-Q2023-884093 Rev 3; broken tack weld does not adversely affect the intended service)	NOT REQUIRED
U2C10-002	F-A	RHRG-2-13-A	LOOSE BOLTS	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (loose bolt was replaced)	NOT REQUIRED
U2C10-003	F-A	RHRG-2-14-D	LOOSE BOLTS	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (loose bolt was replaced)	NOT REQUIRED
U2C10-007	B-K-1	2-47B458S0013-1A	LINEAR INDICATIONS	INDICATION REMOVED PER SSP-7.50 APPENDIX U	YES* 100% EXAMINED
U2C10-008	F-A	2-47B452S0229	SETTING OUT OF RANGE LOOSE BOLT	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (setting acceptable per calc CD-Q2999-990049 Rev 0; loose bolt was tightened)	NOT REQUIRED
U2C10-009	F-A	2-47B452S0234	LOOSE AND MISSING PARTS	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (missing part was replaced and loose bolt tightened)	NOT REQUIRED
U2C10-012	F-A	2-47B408S0076	LOOSE BOLT	REPAIR	YES (7)
U2C10-013	F-A	2-47B452-1320	SETTING OUT OF RANGE	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (setting acceptable per calc CD-Q2999-990049 Rev 0)	NOT REQUIRED



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
 OFFICE OF NUCLEAR POWER PO BOX 2000
 1101 MARKET STREET DECATUR, ALABAMA 35609-2000
 CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

U2C10-014	F-A	2-47B452S0113	TACK WELD BROKEN	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (broken tack weld acceptable per calc CD-Q2999-990049 Rev 0)	NOT REQUIRED
U2C10-015	F-A	2-47B400S0105	LOOSE PISTON	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (snubber replaced)	NOT REQUIRED
U2C10-021	F-A	HPAS-2-H-11	SETTING OUT OF RANGE	EVALUATED ACCEPTABLE AS IS FOR CONT. SERVICE (setting acceptable per calc CD-Q2999-990049 Rev 0)	NOT REQUIRED
U2C10-024	B-G-2	PCV1-2-004-VBC	DAMAGED THREADS	DAMAGED DURING DISASSEMBLY; REPLACED (thread were damage during disassembly; good maintenance practice dictates replacing the bolts)	NOT REQUIRED

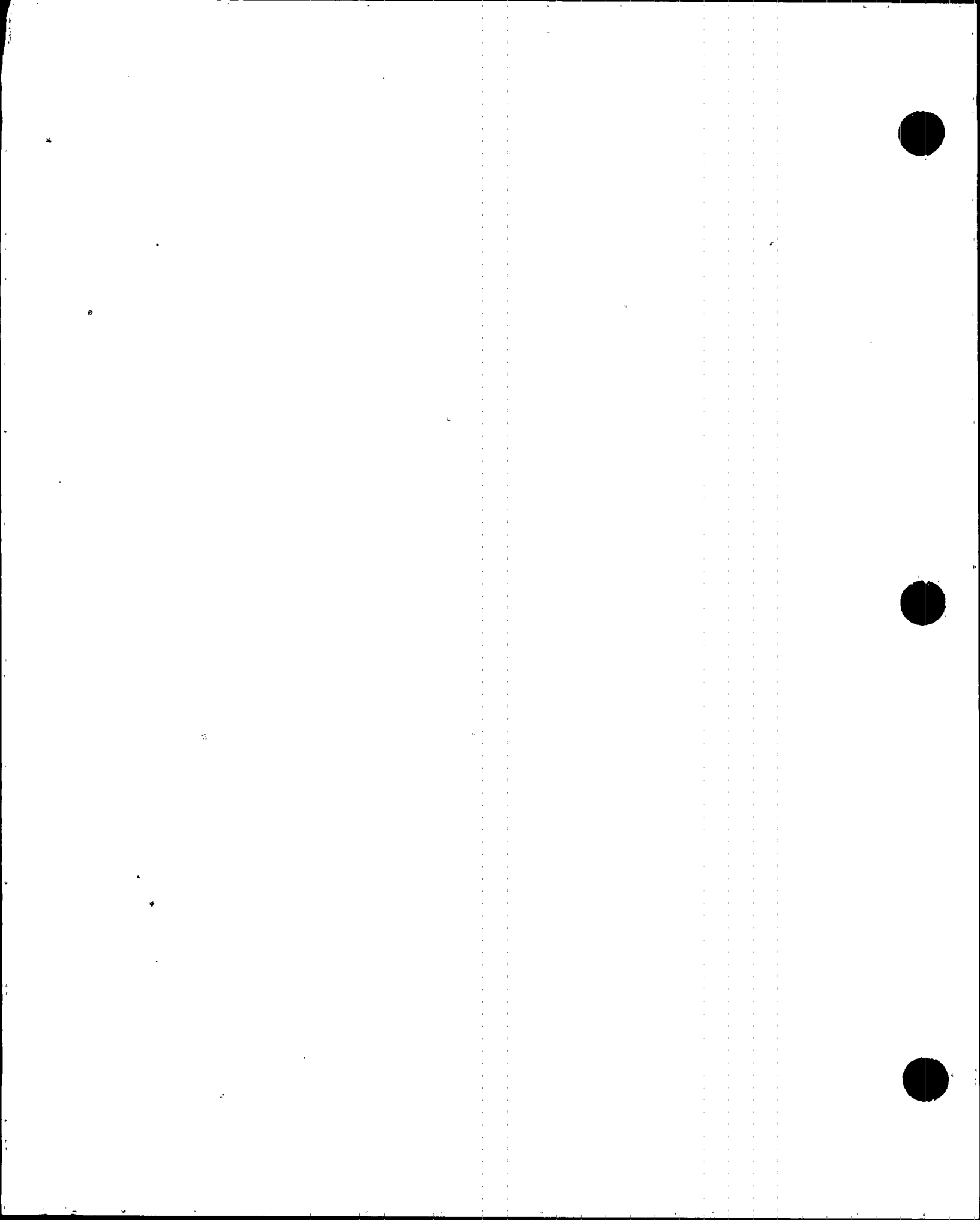
ADDITIONAL SAMPLES

SYSTEM COMPONENT CATEGORY ITEM NO. EXAM METHOD

CORE SPRAY *

*100% population of Core Spray, Category B-K-1 were scheduled to be examined this outage. No additional sample available.

RECIR SYSTEM	2-47B408S0042	F-A	F1.10D	VT-3
RECIR SYSTEM	2-47B408S0059	F-A	F1.10D	VT-3
RECIR SYSTEM	2-47B408S0060	F-A	F1.10D	VT-3
RECIR SYSTEM	2-47B408S0071	F-A	F1.10D	VT-3
RECIR SYSTEM	2-47B408S0075	F-A	F1.10D	VT-3
RECIR SYSTEM	2-47B408S0077	F-A	F1.10D	VT-3
RECIR SYSTEM	2-47B408S0078	F-A	F1.10D	VT-3



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

ATTACHMENT 1

EXAMINATION PLAN (AUGMENTED) AND SUMMARY



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

AUGMENTED EXAMINATION SUMMARY

This section includes information of the augmented examinations performed to comply with NRC or TVA self-imposed requirements. Typical sources include GE Generic Letters, IE Bulletins, Technical Specifications, vendor recommendations, and industry experience. The following summarizes the augmented examinations performed during the unit 2 cycle 10 outage and references the corresponding paragraph in 2-SI-4.6.G.

Paragraph 7.12.3 Welds KR-2-14, KR-2-36, KR-2-37, and KR-2-41

These welds have IGSCC indications that must be monitored for evidence of growth. The examinations performed during cycle 10 revealed no growth. The indications remain essentially unchanged. Also, weld KR-2-37 has a "volumetric" indication that requires monitoring for evidence of change. The "volumetric" indication remains unchanged.

Paragraph 7.12.6 CRD Return Line Reroute (NUREG-0619)

The weld RCRD-2-45 shall be ultrasonically (UT) examined, including the base material on each side of the weld within one wall thickness. The pipe into which the CRD return flow is connected shall be UT examined to a distance of one pipe diameter downstream of the welded connection. Welds RCRDS-2-3 and RCRD-2-44 shall be UT examined along with the pipe on the downstream side. These examinations revealed no evidence of cracking.

Paragraph 7.12.8 Augmented Examination of Austenitic Stainless Steel and Dissimilar Metal Welds Susceptible to IGSCC (Generic Letter 88-01 and NUREG-0313, Revision 2)

Austenitic stainless steel and dissimilar metal circumferential welds in piping four inches or larger NPS that contain reactor coolant at temperatures above 200 degrees F during power operation shall be examined in accordance with Generic Letter 88-01 and NUREG-0313, Revision 2. There was no new IGSCC identified during cycle 10.

NUREG-0313 CATEGORY	TOTAL NUMBER OF WELDS	WELDS EXAMINED DURING U2/C10 Outage
A	46	0
B	N/A	N/A
C	116	23
D	7	4
E	16	9
F	N/A	N/A
G	2	2 (VT-2)



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Paragraph 7.12.9: Technical Surveillance Requirement (TSR) 3.4.3.2

A UT examination shall be performed on welds to locate evidence of cracking due to "pipe-whip". A UT and MT examination was performed on the following welds, TSCS-2-408, GFW-2-09, GFW-2-32, KFW-2-13, THPCI-2-070A, THPCI-2-071, GMS-2-15, and KMS-2-024. A UT and PT examination was performed on welds, DSRWC-2-03(OL), DSRWC-2-05(OL), AND DSRWC-2-06. The examinations were performed to detect evidence of "pipe whip".

The examinations revealed no evidence of cracking due to "pipe-whip".

Paragraph 7.12.11.1 Core Spray Spargers and Associated Piping

The examination requirements for the core spray spargers and associated piping are contained in the plant Surveillance Instruction MSI-0-001-INS001 and/or contractor procedure, which implements IE Bulletin 80-13. The spargers are required to be visually examined each refueling outage. The examination of the core spray spargers was performed during the unit 2 cycle 10 outage.

The examination revealed no reportable indications.

Paragraph 7.12.11.2 Core Support Shroud SIL-572

A visual examination was performed of the Core Support Shroud to the recommendations of SIL-572 and BRWVIP-38. The visual examination was performed on the Shroud to Shroud Support Ledge Weld H-8 from 0 Degrees to 180 Degrees and the RPV to Shroud Support Ledge Weld H-9 from 0 Degrees to 180 Degrees.

The examination revealed no reportable indications.

Paragraph 7.12.11.3 Shroud Support Access Hole Covers SIL-462

Inspect the shroud access hole covers by performing a UT examination of the access hole cover weld area. The UT examination of the access hole covers located at 0 and 180-degrees revealed no reportable indications.

Paragraph 7.12.11.5 Core Spray T-Box Welds SIL-289

A visual examination of Core Spray T-Box to front cover plate weld and the Core Spray T-Box to thermal sleeve weld was performed in conjunction with the NRC IE Bulletin 80-13 examination of the Core Spray Sparger.

The examination revealed no reportable indications.

Paragraph 7.12.11.9 Jet Pump Riser Braces SIL-551

An enhance visual examination (EVT-1) of Riser Braces, #11 through #20 per the recommendations of BRWVIP-41 was performed during the unit 2 cycle 10 outage.

The examination revealed no reportable indications.



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

Paragraph 7.12.11.17 Top Guide and Core Plate Bolting Examination

The top guide and core plate bolting was visually examined during cycle 10. The top guide examination includes the accessible areas of the members which provide the load between the top guide and the shroud.

The examination revealed no reportable indications.

Paragraph 7.11.4 17 Jet Pump Riser Pipe

The jet pump thermal sleeve-to-riser pipe elbow welds were examined during cycle 10. The examination revealed no reportable indications.



EXAM REQUIREMENTS
 A01-02 B01-02
 B02-02 B04-02
 B05-02 D03-02
 P10-02 R27-02
 S01-02 V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
CRDS	2-ISV-085-612/3015		R27-02	B-P	B15.70	VT-1		19990502	R-370	P	4 1/2 X 1 1/8 BLT WO98-014041-007
CRDS	RCRD-2-44	ISI-0272-C 01	B01-02	7126	NU0619	UT	BF-01	19990423	R-256	P	>90% COVERAGE
CRDS	RCRD-2-45	ISI-0272-C 01	B01-02	7126	NU0619	UT	BF-29	19990423	R-261	P	
CRDS	RCRD-2-50	ISI-0272-C 01	B02-02	D	NU0313	UT	BF-29	19990424	R-311	P	~75% COVERAGE ACHIEVED
CRDS	RCRDS-2-03	ISI-0272-C 01	B01-02	7126	NU0619	UT	BF-35	19990423	R-259	P	
CSS	2-47B458S0004	ISI-0280-C 01	P10-02	F-A	F1.10D	VT-3		19990413	R-115	P	AFS 2.875 WR #98-012557-000
CSS	2-CKV-75-0570D		R27-02	B-P	B15.70	VT-1		19990419	R-193	P	REF WO 97-000106-000
CSS	DCS-2-04	ISI-0271-C 01	B02-02	C	NU0313	UT	BF-79	19990413	R-254	P	100% COVERAGE
CSS	DCS-2-07	ISI-0271-C 01	B02-02	C	NU0313	UT	BF-79	19990413	R-146	P	100% COVERAGE
CSS	DSCS-2-02	ISI-0271-C 01	B02-02	C	NU0313	UT	BF-79	19990412	R-144	P	100% COVERAGE
CSS	DSCS-2-09	ISI-0271-C 01	B02-02	C	NU0313	UT	BF-79	19990413	R-145	P	100% COVERAGE
CSS	TCS-2-405	ISI-0271-C 01	B02-02	C	NU0313	UT	BF-51	19990413	R-137	P	~ 97% COVERAGE
CSS	TCS-2-405	ISI-0271-C 01	B02-02	C	NU0313	UT	BF-75	19990413	R-137	P	~ 97% COVERAGE
CSS	TCS-2-406	ISI-0271-C 01	B02-02	C	NU0313	UT	BF-75	19990413	R-136	P	100% COVERAGE
CSS	TCS-2-426	ISI-0271-C 01	B02-02	C	NU0313	UT	BF-75	19990413	R-310	P	ONE SIDE EXAM
CSS	TSCS-2-408	ISI-0271-C 01	B04-02	B-J	TS3432	MT		19990414	R-129	P	> 90% COVERAGE
CSS	TSCS-2-408	ISI-0271-C 01	B04-02	B-J	TS3432	UT	BF-75	19990415	R-148	P	100% COVERAGE
FWS	2-47B415S0004	ISI-0277-C 01	P10-02	F-A	F1.10D	VT-3		19990415	R-142	P	AFS 1.05 WR #98-011553-00
FWS	2-47B415S0012	ISI-0277-C 01	P10-02	F-A	F1.10D	VT-3		19990424	R-116	P	AFS 3.75
FWS	GFW-2-09	ISI-0269-C 01	B04-02	B-J	TS3432	MT		19990419	R-189	P	
FWS	GFW-2-09	ISI-0269-C 01	B04-02	B-J	TS3432	UT	BF-93	19990422	R-240	P	100% COVERAGE
FWS	GFW-2-32	ISI-0269-C 01	B04-02	B-J	TS3432	MT		19990419	R-191	P	



EXAM REQUIREMENTS
 A01-02 B01-02
 B02-02 B04-02
 B05-02 D03-02
 P10-02 R27-02
 S01-02 V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
FWS	GFW-2-32	ISI-0269-C 01	B04-02	B-J	TS3432	UT	BF-93	19990422	R-241	P	100% COVERAGE
FWS	KFW-2-13	ISI-0269-C 01	B04-02	B-J	TS3432	MT		19990419	R-190	P	
FWS	KFW-2-13	ISI-0269-C 01	B04-02	B-J	TS3432	UT	BF-93	19990422	R-239	P	100% COVERAGE
HPCIS	2-47B455R0020	ISI-0130-C 01	P10-02	F-A	F1.20D	VT-3		19990316	R-066	P	AFS 2.0" WR #98-012567-00
HPCIS	2-RTV-073-0225A		R27-02	B-P	B15.70	VT-1		19990502	R-368	P	4 1/2 X 1 1/8-INCH BLT WO98-014041-005
HPCIS	THPCI-2-070A	ISI-0273-C 01	B04-02	B-J	TS3432	MT		19990422	R-234	P	
HPCIS	THPCI-2-070A	ISI-0273-C 01	B04-02	B-J	TS3432	UT	BF-05	19990423	R-258	P	100% COVERAGE
HPCIS	THPCI-2-071	ISI-0273-C 01	B04-02	B-J	TS3432	MT		19990422	R-235	P	
HPCIS	THPCI-2-071	ISI-0273-C 01	B04-02	B-J	TS3432	UT	BF-05	19990423	R-257	P	100% COVERAGE
HPCIS	THPCI-2-074	ISI-0273-C 01	V01-02	B-J	B9.11	MT		19990425	R-309	P	
HPCIS	THPCI-2-074	ISI-0273-C 01	V01-02	B-J	B9.11	UT	BF-05	19990425	R-325	P	100% COVERAGE
MSS	2-47B2401-35	ISI-0412-C 08	P10-02	F-A	F1.20D	VT-3		19990424	R-312	P	AFS 4" WR#98-013404-000
MSS	2-47B400S0005	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990501	R-362	P	
MSS	2-47B400S0006	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990501	R-363	P	
MSS	2-47B400S0007	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990501	R-364	P	
MSS	2-47B400S0008	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990430	R-358	P	RIGID SUPPOR; REINSTALLATION
MSS	2-47B400S0097	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990422	R-249	P	AFS 3.44" WR#98-011537-000
MSS	2-47B400S0103	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990418	R-165	P	AFS 1.75 WR# 98-012655-000
MSS	2-47B400S0104	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990415	R-150	P	AFS 3.375 WR #98-013701-000
MSS	2-47B400S0105	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990422	R-253	P	AFS 4.375 WR#98-013706-000
MSS	2-47B400S0107	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990426	R-330	P	SET 2.875" WR#777
MSS	2-47B400S0108	ISI-0279-C 02	P10-02	F-A	F1.10D	VT-3		19990418	R-166	P	AFS 1.125 WR#98-012651-000



EXAM REQUIREMENTS
 A01-02 B01-02
 B02-02 B04-02
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OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
MSS	2-47B400S0109	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990425	R-305	P	AFS 1.875" WR#98-012621-000
MSS	2-47B400S0110	ISI-0279-C 01	P10-02	F-A	F1.10D	VT-3		19990416	R-149	P	AFS 2.875" WR #98-011689-000
MSS	FCV-01-015	ISI-0222-C 01	P10-02	B-G-2	B7.70	VT-1		19990423	R-265	P	1- BODY STUD & 2 NUTS, WR#97006387-068
MSS	FCV-01-015	ISI-0222-C 01	P10-02	B-M-2	B12.50	VT-3		19990418	R-164	P	BONNET/VALVE S/N 4 WR#98-013825-000
MSS	FCV-01-015-BC	ISI-0222-C 01	P10-02	B-G-2	B7.70	VT-1		19990419	R-196	P	PSI ON ONE NEW STUD
MSS	FCV-01-038	ISI-0222-C 02	P10-02	B-M-2	B12.50	VT-3		19990418	R-163	P	BONNET/VALVE COVER S/N 3
MSS	GMS-2-15	ISI-0222-C 02	B04-02	B-J	TS3432	MT		19990415	R-141	P	
MSS	GMS-2-15	ISI-0222-C 02	B04-02	B-J	TS3432	UT	BF-104	19990416	R-159	P	100% COVERAGE
MSS	HPAS-2-H-11	ISI-0079-C 02	P10-02	F-A	F1.20D	VT-3		19990503	R-367	P	AFS 913# WR#99-000239-000
MSS	KMS-2-024	ISI-0222-C 01	B04-02	B-J	TS3432	MT		19990415	R-140	P	
MSS	KMS-2-024	ISI-0222-C 01	B04-02	B-J	TS3432	UT	BF-104	19990416	R-161	P	100% COVERAGE
MSS	PCV1-2-004	ISI-0312-B 01	P10-02	B-G-2	B7.70	VT-1		19990423	R-266	P	12 NEW STUDS ONLY WO#98-013558-000
MSS	PCV1-2-004-VBC	ISI-0312-B 01	V01-02	B-G-2	B7.70	VT-1		19990428	R-344	F	VALVE S/N#1016 WO#98-013558-001
MSS	PCV1-2-023	ISI-0312-B 01	P10-02	B-G-2	B7.70	VT-1		19990423	R-267	P	12NEWSTUDS24NEWNUTS WO#98-013625-000
MSS	PCV1-2-023	ISI-0312-B 01	P10-02	B-M-2	B12.50	VT-3		19990424	R-264	P	WR# 98-013625-000
RCICS	2-47B456-116	ISI-0131-C 01	P10-02	F-A	F1.20D	VT-3		19990319	R-072	P	AFS 3.85 WR #98-002774
RCICS	2-47B456R0007	ISI-0131-C 01	P10-02	F-A	F1.20D	VT-3		19990505	R-372	P	AFS 4-3/4" (4" DIA P) WR#98-012564-002
RCICS	2-RTV-071-0001-BC		R27-02	B-P	B15.70	VT-1		19990502	R-369	P	4 1/2 X 1 1/8-INCH BLT WO98-014041-006
RCICS	RCIC-2-009-002	ISI-0129-C 01	P10-02	B-J	B9.11	PT		19990401	R-111	P	
RCICS	RCIC-2-009-003	ISI-0129-C 01	P10-02	B-J	B9.11	PT		19990324	R-092	P	
RCICS	RCIC-2-009-004	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-262	P	
RCICS	RCIC-2-009-009	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-268	P	



EXAM REQUIREMENTS

A01-02 B01-02
 B02-02 B04-02
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 P10-02 R27-02
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 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	2-47B408S0042	ISI-0278-C 02	A01-02	F-A	F1.10C	VT-3		19990422	R-230	P	AFS 1-13/16"
RECIR	2-47B408S0046	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990419	R-187	P	AFS 4.25" WR#98-013724-000
RECIR	2-47B408S0047	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990501	R-361	P	AFS 2-1/2" WR#98-013727-000
RECIR	2-47B408S0048	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990501	R-360	P	AFS 3-1/2" WR#98-012807-000
RECIR	2-47B408S0053-IE	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990427	R-322	P	AFS 2.562" WR#98-012927-000
RECIR	2-47B408S0054-IE	ISI-0278-C 02	P10-02	F-A	F1.40D	VT-3		19990427	R-320	P	AFS 1.312" WR#98-012910-000
RECIR	2-47B408S0058	ISI-0278-C 01	P10-02	F-A	F1.10D	VT-3		19990430	R-357	P	AFS 1-3/4" WR#98-012685
RECIR	2-47B408S0059	ISI-0278-C 01	A01-02	F-A	F1.10C	VT-3		19990422	R-231	P	AFS 0.9 INCREMENTS
RECIR	2-47B408S0060	ISI-0278-C 01	A01-02	F-A	F1.10C	VT-3		19990422	R-226	P	AFS 4-3/8 & 4-3/8"
RECIR	2-47B408S0061	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990424	R-263	P	AFS 4.5" WR#98-013723-000
RECIR	2-47B408S0062	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990429	R-351	P	AFS 3.75" WR#98-013805-000
RECIR	2-47B408S0063	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990429	R-336	P	SET 3.75" WR#98-013726-000
RECIR	2-47B408S0067-IE	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990427	R-338	P	SET 2.6" WR#98-012877-000
RECIR	2-47B408S0068-IE	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990429	R-355	P	AFS 0.375" WR#98-012911-000
RECIR	2-47B408S0069-IE	ISI-0278-C 01	P10-02	F-A	F1.40D	VT-3		19990427	R-340	P	AFS 2.25" WR#98-012873-000
RECIR	2-47B408S0071	ISI-0278-C 01	A01-02	F-A	F1.10D	VT-3		19990422	R-228	P	AFS 3.5"
RECIR	2-47B408S0071	ISI-0278-C 01	P10-02	F-A	F1.10D	VT-3		19990425	R-304	P	AFS 4.25" WR#98-01378-000
RECIR	2-47B408S0073	ISI-0278-C 01	P10-02	F-A	F1.10D	VT-3		19990422	R-206	P	AFS 1.75" WR#98-013024-000
RECIR	2-47B408S0075	ISI-0278-C 02	A01-02	F-A	F1.10	VT-3		19990421	R-218	P	AFS 10810#
RECIR	2-47B408S0076	ISI-0278-C 02	P10-02	F-A	F1.10D	VT-3		19990428	R-342	P	AFS 2.75" WR#98-013730-000
RECIR	2-47B408S0077	ISI-0278-C 02	A01-02	F-A	F1.10C	VT-3		19990421	R-217	P	AFS 7250#
RECIR	2-47B408S0078	ISI-0278-C 02	A01-02	F-A	F1.10C	VT-3		19990422	R-227	P	AFS 2-11/16"



EXAM REQUIREMENTS
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 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
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System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	2-47B408S0078	ISI-0278-C 02	P10-02	F-A	F1.10D	VT-3		19990429	R-356	P	AFS 3.5" WR#98-0137731-000
RECIR	2-47B408S0081	ISI-0278-C 01	P10-02	F-A	F1.10D	VT-3		19990428	R-345	P	AFS 2-3/16" WR#98-013824-000
RECIR	2-47B408S0082	ISI-0278-C 02	P10-02	F-A	F1.10D	VT-3		19990415	R-152&1	P	AFS 3.95&1.7 WR #98-013825(28)-000
RECIR	2-47B452S0237	ISI-0278-C 02	P10-02	F-A	F1.10D	VT-3		19990427	R-339	P	AFS 2.375" WR#98-012561-000
RECIR	GR-2-12	ISI-0270-C 01	B02-02	C	NU0313	UT	BF-79	19990418	R-237	P	100% COVERAGE
RECIR	GR-2-15(OL)	ISI-0270-C 01	B02-02	E	NU0313	UT	BF-50	19990420	R-213	P	PREVIOUSLY RECORDED INDICATION
RECIR	GR-2-54	ISI-0270-C 01	B02-02	C	NU0313	UT	BF-57	19990428	R-354	P	100% COVERAGE
RECIR	GR-2-60	ISI-0270-C 02	B02-02	C	NU0313	UT	BF-57	19990417	R-208	P	100% COVERAGE
RECIR	GR-2-64(OL)	ISI-0270-C 02	B02-02	E	NU0313	UT	BF-83	19990421	R-220	P	OVERLAY WELD;
RECIR	KR-2-02	ISI-0270-C 01	B02-02	C	NU0313	UT	BF-57	19990421	R-222	P	100% COVERAGE
RECIR	KR-2-12	ISI-0270-C 01	B02-02	C	NU0313	UT	BF-57	19990419	R-211	P	~75% COVERAGE ACHIEVED
RECIR	KR-2-14	ISI-0270-C 01	D03-02	E	NU0313	UT	BF-88	19990423	R-260	P	IGSCC REMAINS UNCHANGED
RECIR	KR-2-33	ISI-0270-C 02	B02-02	C	NU0313	UT	BF-44	19990419	R-212	P	~80% COVERAGE ACHIEVED
RECIR	KR-2-36	ISI-0270-C 02	D03-02	E	NU0313	UT	BF-88	19990418	R-168	P	IGSCC REMAINS UNCHANGED
RECIR	KR-2-37	ISI-0270-C 02	D03-02	E	NU0313	UT	BF-88	19990418	R-188	P	IGSCC REMAINS UNCHANGED
RECIR	KR-2-41	ISI-0270-C 02	D03-02	E	NU0313	UT	BF-88	19990420	R-255	P	IGSCC REMAINS UNCHANGED
RECIR	KR-2-45	ISI-0270-C 01	B02-02	C	NU0313	UT	BF-57	19990417	R-209	P	100% COVERAGE
RECIR	KR-2-46	ISI-0270-C 01	B02-02	C	NU0313	UT	BF-57	19990420	R-238	P	ONE SIDE EXAMINATION
RECIR	KR-2-47	ISI-0270-C 01	B02-02	C	NU0313	UT	BF-57	19990421	R-221	P	100% COVERAGE
RECIR	KR-2-51	ISI-0270-C 02	B02-02	C	NU0313	UT	BF-57	19990418	R-210	P	100% COVERAGE
RECIR	PMP-A-STUD-2-01	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-02	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	



EXAM REQUIREMENTS

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 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	PMP-A-STUD-2-03	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-04	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-05	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-06	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-07	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-08	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-09	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-10	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-11	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-12	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-13	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-14	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-15	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-16	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	
RECIR	PMP-A-STUD-2-16	ISI-0407-C 01	V01-02	B-G-1	B6.180	VT-1		19990421	R-202	P	STUDS EXAMINED IN PLACE
RHRS	2-47B452-1206	ISI-0324-C 10	P10-02	F-A	F1.20D	VT-3		19990503	R-366	P	AFS 3-1/8" WEST SNB WR#99-003435-001
RHRS	2-47B452-1320	ISI-0324-C 12	P10-02	F-A	F1.20D	VT-3		19990503	R-365	P	AFS 8366#
RHRS	2-47B452-416	ISI-0324-C 11	P10-02	F-A	F1.20D	VT-3		19990507	R-374	P	AFS 3-1/16" WR#99-003436-034
RHRS	2-47B452R0051	ISI-0324-C 04	P10-02	F-A	F1.20D	VT-3		19990505	R-373	P	AFS 2.95" WR#99-003436-017
RHRS	2-47B452S0157	ISI-0324-C 08	R16-02	F-A	F1.20D	VT-3		19990409	R-114	P	AFS 4.4 WR #98-13816
RHRS	2-47B452S0227	ISI-0276-C 01	P10-02	F-A	F1.10D	VT-3		19990423	R-247	P	AFS 1.375" WR#98-013698-000
RHRS	2-47B452S0235	ISI-0276-C 01	P10-02	F-A	F1.10D	VT-3		19990428	R-341	P	AFS 2.43" WR#98-012563-000



EXAM REQUIREMENTS
 A01-02 B01-02
 B02-02 B04-02
 B05-02 D03-02
 P10-02 R27-02
 S01-02 V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	2-47B452S0240	ISI-0276-C 01	P10-02	F-A	F1.10D	VT-3		19990423	R-246	P	AFS 1.437" WR#98-013696-000
RHRS	2-47B452S0250	ISI-0324-C 09	P10-02	F-A	F1.20D	VT-3		19990330	R-099	P	AFS 3.9" WR #98-002770
RHRS	2-47B452S0251	ISI-0324-C 09	P10-02	F-A	F1.20D	VT-3		19990509	R-375	P	AFS 4-5/8" WR#99-003437-001
RHRS	2-SHV-074-0754		R27-02	B-P	B15.70	VT-1		19990502	R-371	P	NEW BOLTING WO 98-014041-004
RHRS	DRHR-2-03	ISI-0221-C 01	B02-02	D	NU0313	UT	BF-57	19990401	R-105	P	~85% COVERAGE CONFIGURATION
RHRS	DRHR-2-03B	ISI-0221-C 01	B02-02	G	NU0313	VT-2		19990502	R-376	P	
RHRS	DRHR-2-04	ISI-0221-C 01	B02-02	C	NU0313	UT	BF-57	19990416	R-155	P	100% COVERAGE
RHRS	DRHR-2-06	ISI-0221-C 01	B02-02	C	NU0313	UT	BF-57	19990415	R-158	P	100% COVERAGE
RHRS	DRHR-2-11	MSG-0018-C 09	B02-02	D	NU0313	UT	BF-102	19990402	R-110	P	>90% COVERAGE
RHRS	DRHR-2-13B	ISI-0221-C 01	B02-02	G	NU0313	VT-2		19990502	R-376	P	
RHRS	DRHR-2-16	ISI-0221-C 01	B02-02	C	NU0313	UT	BF-57	19990415	R-157	P	100% COVERAGE
RHRS	DRHR-2-21	ISI-0221-C 01	B02-02	C	NU0313	UT	BF-87	19990416	R-156	P	100% COVERAGE
RHRS	DSRHR-2-01	ISI-0221-C 01	B02-02	C	NU0313	UT	BF-57	19990416	R-154	P	100% COVERAGE
RHRS	RHRG-2-12-A	ISI-0406-C 01	S01-02	F-A	F1.40B	VT-3		19990311	R-050	P	SUCCESSIVE EXAM
RHRS	RHRG-2-12-C	ISI-0406-C 01	S01-02	F-A	F1.40B	VT-3		19990311	R-049	P	SUCCESSIVE EXAM
RHRS	RHRG-2-13-A	ISI-0406-C 01	S01-02	F-A	F1.40B	VT-3		19990311	R-048	F	SUCCESSIVE EXAM
RHRS	RHRG-2-14-D	ISI-0406-C 01	S01-02	F-A	F1.40B	VT-3		19990317	R-069	F	SUCCESSIVE EXAM
RPV	ACCHCOV-1	N/A 00	B06-02	RPV	INT	UT		19990423	R-250	P	0-DEGREE ACCESS HOLE COVER
RPV	ACCHCOV-2	N/A 00	B06-02	RPV	INT	UT		19990423	R-250	P	180-DEGREE ACCESS HOLE COVER
RPV	JPRISBR-2-11	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	JPRISBR-2-12	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	JPRISBR-2-13	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	



EXAM REQUIREMENTS
 A01-02 B01-02
 B02-02 B04-02
 B05-02 D03-02
 P10-02 R27-02
 S01-02 V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	JPRISBR-2-14	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	JPRISBR-2-15	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	JPRISBR-2-16	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	JPRISBR-2-17	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	JPRISBR-2-18	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	JPRISBR-2-19	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	JPRISBR-2-20	N/A 00	B06-02	RPV	INT	VT-3		19990424	R-251	P	
RPV	RCH-2-2C-FLEX	ISI-0408-C 01	V01-02	B-A	B1.40	MT		19990422	R-219	P	VOL. PERFORMED ON INSIDE SURFACE
RPV	RCRD-2-33	ISI-0272-C 01	B02-02	D	NU0313	UT	BF-76	19990425	R-324	P	DISIMILAR METAL WELD
RPV	RCRD-2-33	ISI-0272-C 01	B02-02	D	NU0313	UT	BF-60	19990425	R-324	P	DISSIMILAR METAL WELD
RPV	RPV-INTERIOR	CHM-2046-C 02	B06-02	RPV	712111	EVT		19990424	R-251	P	SEE REPORT FOR ITEMS EXAMINED
RPV	RPV-INTERIOR	CHM-2046-C 02	B06-02	RPV	712112	EVT		19990424	R-251	P	SEE REPORT FOR ITEMS EXAMINED
RPV	RPV-INTERIOR	CHM-2046-C 02	B06-02	RPV	712115	EVT		19990424	R-251	P	SEE REPORT FOR ITEMS EXAMINED
RPV	RPV-INTERIOR	CHM-2046-C 02	B06-02	RPV	712119	EVT		19990424	R-251	P	SEE REPORT FOR ITEMS EXAMINED
RPV	TOPGUIDE	N/A 00	B06-02	RPV	INT	VT		19990424	R-251	P	
RWCU	DSRWC-2-03(OL)	ISI-0272-C 01	B04-02	B-J	TS3432	PT		19990427	R-332	P	
RWCU	DSRWC-2-03(OL)	ISI-0272-C 01	B04-02	B-J	TS3432	UT	BF-62	19990427	R-326	P	95% COVERAGE ACHIEVED
RWCU	DSRWC-2-03(OL)	ISI-0272-C 01	B02-02	E	NU0313	UT	BF-80	19990427	R-326	P	~95% COVERAGE ACHIEVED
RWCU	DSRWC-2-04(OL)	ISI-0272-C 01	B02-02	E	NU0313	UT	BF-80	19990427	R-327	P	~96% COVERAGE ACHIEVED
RWCU	DSRWC-2-05(OL)	ISI-0272-C 01	B04-02	B-J	TS3432	PT		19990427	R-333	P	
RWCU	DSRWC-2-05(OL)	ISI-0272-C 01	B04-02	B-J	TS3432	UT	BF-80	19990427	R-328	P	94% COVERAGE ACHIEVED
RWCU	DSRWC-2-05(OL)	ISI-0272-C 01	B02-02	E	NU0313	UT	BF-62	19990427	R-328	P	~94% COVERAGE ACHIEVED



EXAM REQUIREMENTS
 A01-02 B01-02
 B02-02 B04-02
 B05-02 D03-02
 P10-02 R27-02
 S01-02 V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

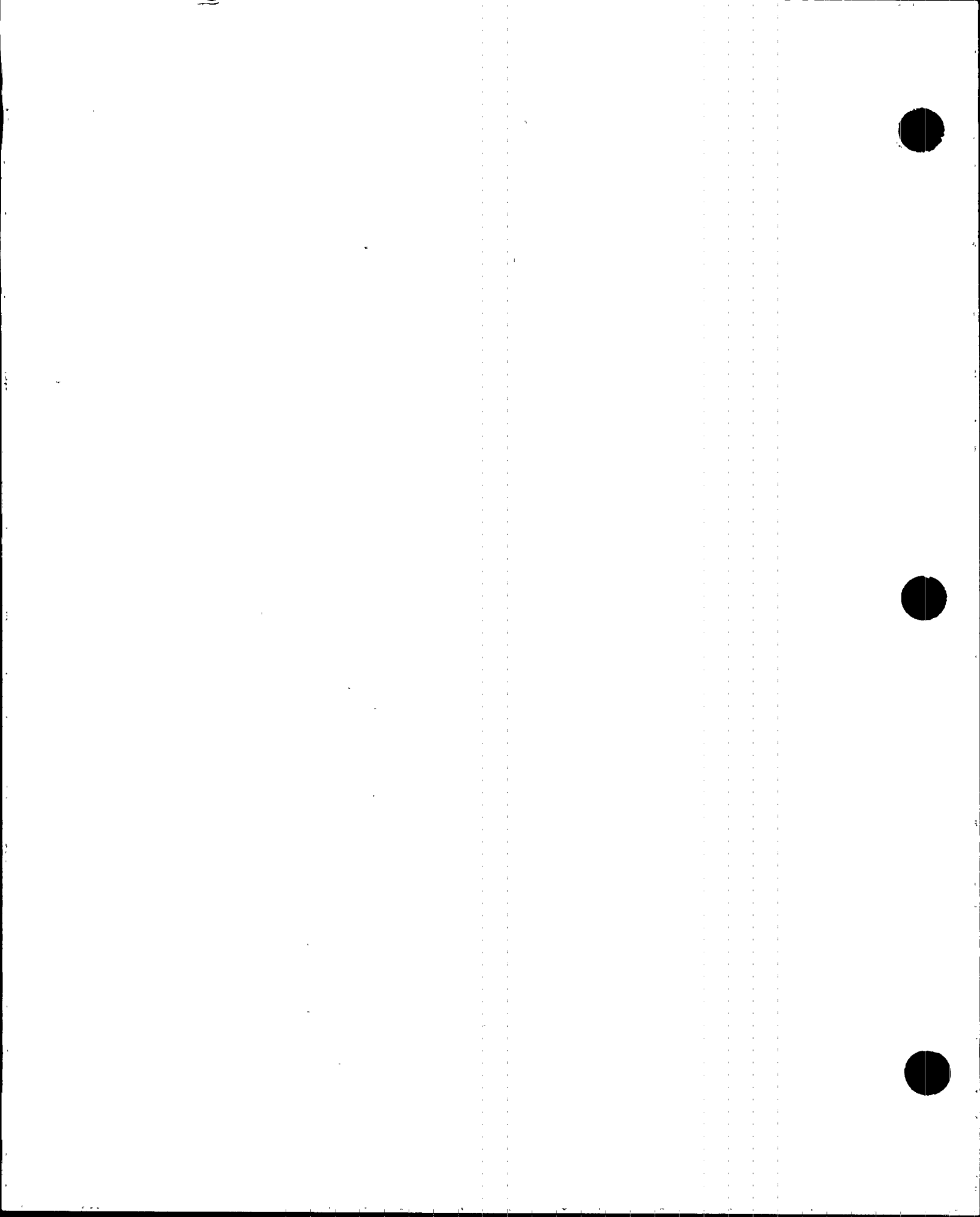
PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RWCU	DSRWC-2-06	ISI-0272-C 01	B04-02	B-J	TS3432	PT		19990427	R-334	P	
RWCU	DSRWC-2-06	ISI-0272-C 01	B04-02	B-J	TS3432	UT	BF-31	19990427	R-323	P	100% COVERAGE
RWCU	DSRWC-2-06	ISI-0272-C 01	B02-02	C	NU0313	UT	BF-31	19990427	R-323	P	100% COVERAGE
RWCU	FCV-69-001	ISI-0272-C 01	P10-02	B-M-2	B12.50	VT-3		19990406	R-107	P	ACCESSABLE INTERNAL SURFACES
RWCU	FCV-69-002	ISI-0272-C 01	P10-02	B-M-2	B12.50	VT-3		19990406	R-108	P	
RWCU	RWCU-2-003-025	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-306	P	AFTER RT REPAIR
RWCU	RWCU-2-003-025	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990422	R-223	P	
RWCU	RWCU-2-003-025	ISI-0272-C 01	P10-02	B-J	B9.11	UT	BF-61	19990425	R-315	P	100% COVERAGE NEW WELD
RWCU	RWCU-2-003-026	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990331	R-112	P	
RWCU	RWCU-2-003-026	ISI-0272-C 01	P10-02	B-J	B9.11	UT		19990405	R-109	P	
RWCU	RWCU-2-003-027	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-307	P	AFTER REPAIR
RWCU	RWCU-2-003-027	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990426	R-314	P	AFTER REPAIR
RWCU	RWCU-2-003-027	ISI-0272-C 01	P10-02	B-J	B9.11	UT	BF-61	19990426	R-316	P	100% COVERAGE NEW WELD
RWCU	RWCU-2-003-037	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990428	R-346	P	
RWCU	RWCU-2-003-038	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990428	R-346	P	
RWCU	RWCU-2-003-039	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-094	P	
RWCU	RWCU-2-003-039	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990315	R-347	P	DUPLICATE OF REPORT R-094
RWCU	RWCU-2-003-040	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-095	P	
RWCU	RWCU-2-003-040	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-348	P	DUPLICATE OF REPORT R-095
RWCU	RWCU-2-003-041	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990428	R-346	P	
RWCU	RWCU-2-003-042	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-096	P	
RWCU	RWCU-2-003-042	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-349	P	DUPLICATE OF REPORT R-096



EXAM REQUIREMENTS
 A01-02 B01-02
 B02-02 B04-02
 B05-02 D03-02
 P10-02 R27-02
 S01-02 V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 PO BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: TWO CYCLE: 10

COMMERCIAL SERVICE DATE: MARCH 1, 1975

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RWCU	RWCU-2-003-043	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-350	P	DUPLICATE OF REPORT R-093
RWCU	RWCU-2-003-043	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990323	R-093	P	
RWCU	RWCU-2-003-044	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990424	R-308	P	
RWCU	RWCU-2-003-044	ISI-0272-C 01	P10-02	B-J	B9.11	PT		19990426	R-313	P	
RWCU	RWCU-2-003-044	ISI-0272-C 01	P10-02	B-J	B9.11	UT	BF-61	19990426	R-317	P	100% COVERAGE NEWWELD



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

ATTACHMENT 2

APPENDIX IWE - BFN CONTAINMENT INSERVICE INSPECTION



BROWNS FERRY NUCLEAR PLANT
CONTAINMENT INSERVICE INSPECTION PROGRAM
UNIT 2 CYCLE 10 SUMMARY REPORT



BFN CONTAINMENT INSERVICE INSPECTION PROGRAM
U2C10 SUMMARY REPORT

Unit: BFN Unit 2
Refueling Outage: U2C10
Period/Interval: First Period of the First Interval
Code of Record: ASME Section XI, 1992 Edition/1992 Addenda
Program Procedure: 0-TI-376 Revision 0

Summary of Examinations

The records contained within the Unit 2 Cycle 10 Plant Report comprise the initial Containment Inservice examinations performed to implement the requirements of ASME Section XI Subsection IWE. The examinations are summarized as follows:

Table IWE-2500-1, Examination Category E-A, Containment Surfaces

The General Visual Examination of accessible containment surface areas was completed. This examination was performed by procedure 2-TI-173; a copy of the examination is incorporated in the Unit 2 Cycle 10 Plant Report. This completes all examinations required by Category E-A for the First Period.

Table IWE-2500-1, Examination Category E-C, Containment Surfaces Requiring Augmented Examination

100% of the Pressure Suppression Chamber air/water interface area was examined. This is the only area currently identified as an augmented area for BFN Unit 2. This completes all examinations required by Category E-C for the First Period.

Table IWE-2500-1, Examination Category E-D, Seals, Gaskets, and Moisture Barriers

16 of 52 seals (31%) were examined. In addition, Pre-service examinations were performed on all seals replaced.

Table IWE-2500-1, Examination Category E-G, Pressure Retaining Bolting

7 of 25 bolted connections (28%) were examined. These were all the bolted connections that were disassembled during the outage.

Table IWE-2500-1, Examination Category E-P, All Pressure Retaining Components

Appendix J Type B testing was completed on the Drywell Personnel Airlock in accordance with the Containment Leak Rate Program and 2-SR-3.6.1.2.1.

Appendix J Type B testing was completed on the Containment seals and gaskets in accordance with the Containment Leak Rate Program and 2-SI-4.7.A.2.g-2/Fha and 2-SI-4.7.A.2.g-3/74h.

VT-2 examinations were completed for bolting replaced on the drywell head and equipment hatch 2-X-1B.



BFN CONTAINMENT INSERVICE INSPECTION PROGRAM
U2C10 SUMMARY REPORT

TVA Self-Imposed Examinations

UT thickness measurements were taken in the drywell sand bed region to obtain data for determination of Table IWE-2500-1 augmented examination requirements. The data indicated that the condition of the drywell steel liner plate in this area is good, and that this area should not be categorized for augmented examination. This information exam (WO 98-015937-000) is included in the Unit 2 Cycle 10 Plant Report.

UT thickness measurements were taken on the ECCS Ring Header to obtain data for determination of Table IWE-2500-1 augmented examination requirements. The data indicated that the condition of the ECCS Ring Header is good, and that this area should not be categorized for augmented examination. This information exam (WR C384651/WO 99-006957-000) is included in the Unit 2 Cycle 10 Plant Report.

Visual examination of the drywell liner below the moisture seal barrier (MSB) was performed in areas where the MSB was excavated for replacement.



APPENDIX IWE

BFN CONTAINMENT INSERVICE INSPECTION PROGRAM

ASME SECTION XI SUBSECTION IWE

The information contained in this Appendix is provided in accordance with the requirements of 10CFR 50.55a(b)(2)(x)(A), evaluation of inaccessible areas, and 10CFR 50.55a(b)(2)(x)(D), evaluation for additional examinations, as they pertain to containment inservice examinations performed during the BFN Unit 2 Cycle 10 refueling outage.

The subject examinations were performed in accordance with ASME Section XI Subsection IWE, 1992 Edition/1992 Addenda. BFN Unit 2 is in the first period of the first examination interval.



APPENDIX IWE

Report No: CISI-042

Component: Pressure Suppression Chamber
Exterior Surface

Condition/Indication: Indications were noted during the VT-1 examination of the Pressure Suppression Chamber (PSC) Exterior Surface air/water interface region.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coating discontinuities, including minor scratches, scrapes, flaking streaks, and loss of coating to the base metal. The condition of the base metal is good, with no notable signs of material loss and minor corrosion noted in a few of the areas of coatings loss. The conditions are expected conditions based on age and service condition of the component.

(2) Evaluation of each area, and the result of the evaluation:

The conditions noted do not impact the structural integrity or leak tightness of the PSC, and are expected conditions based on the age and service condition of the component. The primary reason this area is being inspected is due to the conditions which could potentially affect the interior surface. There is no accelerated degradation exhibited by this component. Therefore, no degradation exists which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coating discontinuities, including minor scratches, scrapes, flaking streaks, and loss of coating to the base metal. The condition of the base metal is good, with no notable signs of material loss and minor corrosion noted in a few of the areas of coatings loss. The conditions are expected conditions based on age and service condition of the component.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The degradation is not a defective condition with respect to the component function of pressure boundary or structural integrity. Therefore, additional examinations are not warranted.

(3) A description of the necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

100% of the PSC exterior surface was examined in the air/water interface area. Based on the above evaluation, no additional examinations are warranted.



APPENDIX IWE

Report No: CISI-043

Component: Penetration 2-X-46

Condition/Indication: Indications were noted during the VT-3 examination of 2-X-46 prior to modification by DCN T41156B. This inspection was performed to satisfy the requirement for examination prior to coating removal per IWE-2500(b).

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The indications consist of chipped paint on the face of the penetration. No corrosion was noted in the areas of coating discontinuities. The conditions are expected conditions based on age and service condition of the component.

(2) Evaluation of each area, and the result of the evaluation:

The conditions noted do not impact the structural integrity or leak tightness of the drywell, and are expected conditions based on the age and service condition of the component. The subject indications have been evaluated to be acceptable for continued service without repair. Therefore, no degradation exists which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

The indications consist of chipped paint on the face of the penetration. No corrosion was noted in the areas of coating discontinuities. The conditions are expected conditions based on age and service condition of the component.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The degradation is not a defective condition with respect to the component function of pressure boundary or structural integrity. Therefore, additional examinations are not warranted.

(3) A description of the necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

No additional examinations are warranted.



APPENDIX IWE

Report No: CISI-044

Component: Penetration 2-X-43

Condition/Indication: Indications were noted during the VT-3 examination of 2-X-43 prior to modification by DCN T41156B. This inspection was performed to satisfy the requirement for examination prior to coating removal per IWE-2500(b).

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The indications consist of coating discontinuities, minor to localized heavy corrosion, flaking rust, and slight pitting. The conditions are expected conditions based on age and service condition of the component.

(2) Evaluation of each area, and the result of the evaluation:

The conditions noted do not impact the structural integrity or leak tightness of the drywell, and are expected conditions based on the age and service condition of the component. The subject indications have been evaluated to be acceptable for continued service without repair. Therefore, no degradation exists which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

The indications consist of coating discontinuities, minor to localized heavy corrosion, flaking rust, and slight pitting. The conditions are expected conditions based on age and service condition of the component.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The degradation is not a defective condition with respect to the component function of pressure boundary or structural integrity. Therefore, additional examinations are not warranted.

(3) A description of the necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

No additional examinations are warranted.



APPENDIX IWE

Report No: CISI-052

Component: Drywell Equipment Hatch
2-X-1B (Bolting)

Condition/Indication: During VT-1 examination, 3 of 12 bolts were found to have damaged threads.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

Significant physical damage to the threads of the bolts was noted. These bolts do not perform a pressure retaining function. The indications are primarily due to age and repeated handling during the disassembly/reassembly process, and had no impact on the functional capability of the component

(2) Evaluation of each area, and the result of the evaluation:

No degradation was noted which could be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective action necessary, the condition was determined not to be a defective condition. The engineering evaluation concluded that the bolts could be replaced as a maintenance good practice, and they were subsequently replaced.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

Significant physical damage to the threads of the bolts was noted. These bolts do not perform a pressure retaining function. The indications are primarily due to age and repeated handling during the disassembly/reassembly process, and had no impact on the functional capability of the component

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The condition had no impact on the structural integrity or sealing capability of the component, and was determined not to be a defective condition. Therefore, additional examinations are not warranted.

(3) A description of the necessary corrective actions:

No corrective action necessary, the condition was determined not to be a defective condition. The engineering evaluation concluded that the bolts could be replaced as a maintenance good practice, and they were subsequently replaced.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

No additional examinations required.



APPENDIX IWE

Report No: CISI-054

Component: Drywell Head Flange Bolts

Condition/Indication: During VT-1 examination, 66 of the 208 bolt/nut sets had indications requiring engineering evaluation. All 208 drywell head flange bolts and nuts were examined.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

Galling of the head mating surface, physical damage, thread damage, and cracks were noted. These indications did not affect the sealing capability of the bolting. These indications are due to the age of the bolting and the repeated handling during disassembly/reassembly of the drywell head. Additionally, 1 bolt was cut off during disassembly to facilitate removal of the drywell head.

(2) Evaluation of each area, and the result of the evaluation:

The noted conditions were not defective conditions with respect to the structural integrity or sealing capability of the component, excepting the bolt which was purposely cut during removal. Therefore, no defective conditions exist which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

The bolt cut out during head removal was replaced as a Section XI Repair/Replacement. No other corrective actions were warranted.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

Galling of the head mating surface, physical damage, thread damage, and cracks were noted. These indications did not affect the sealing capability of the bolting. These indications are due to the age of the bolting and the repeated handling during disassembly/reassembly of the drywell head. Additionally, 1 bolt was cut off during disassembly to facilitate removal of the drywell head.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

None of the subject flaws affected the structural integrity or sealing capability of the component. The bolt which was cut was not a service-related problem. The indications were evaluated not to be defects. Therefore, additional examinations are not warranted.

(3) A description of the necessary corrective actions:

The bolt cut out during head removal, one bolt due to excessive galling, and others as good maintenance practice were replaced as a Section XI Repair/Replacement. No other corrective actions were warranted.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

No additional examinations required.



APPENDIX IWE

Report No: CISI-055

Component: Pressure Suppression Chamber
Exterior Surface

Condition/Indication: Indications were noted during the VT-1 examination of the Pressure Suppression Chamber (PSC) Exterior Surface air/water interface region.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coating discontinuities, including minor scratches, scrapes, flaking streaks, and loss of coating to the base metal. In the areas where loss of protective coatings occurred, base metal had only minor localized corrosion with no notable signs of significant material loss. The conditions are expected conditions based on age and service condition of the component.

(2) Evaluation of each area, and the result of the evaluation:

The conditions noted do not impact the structural integrity or leak tightness of the PSC, and are expected conditions based on the age and service condition of the component. The primary reason this area is being inspected is due to the conditions which could potentially affect the interior surface. There is no accelerated degradation exhibited by this component. Therefore, no degradation exists which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coating discontinuities, including minor scratches, scrapes, flaking streaks, and loss of coating to the base metal. In the areas where loss of protective coatings occurred, base metal had only minor localized corrosion with no notable signs of significant material loss. The conditions are expected conditions based on age and service condition of the component.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The degradation is not a defective condition with respect to the component function of pressure boundary or structural integrity. Therefore, additional examinations are not warranted.

(3) A description of the necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

100% of the PSC exterior surface was examined in the air/water interface area. Based on the above evaluation, no additional examinations are warranted.



APPENDIX IWE

Report No: CISI-056

Component: Pressure Suppression Chamber
Exterior Surface

Condition/Indication: Indications were noted during the VT-1 examination of the Pressure Suppression Chamber (PSC) Exterior Surface air/water interface region.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coating discontinuities, including minor scratches, scrapes, flaking streaks, and loss of coating to the base metal. In the areas where loss of protective coatings occurred, base metal had only minor localized corrosion with no notable signs of significant material loss. The conditions are expected conditions based on age and service condition of the component.

(2) Evaluation of each area, and the result of the evaluation:

The conditions noted do not impact the structural integrity or leak tightness of the PSC, and are expected conditions based on the age and service condition of the component. The primary reason this area is being inspected is due to the conditions which could potentially affect the interior surface. There is no accelerated degradation exhibited by this component. Therefore, no degradation exists which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coating discontinuities, including minor scratches, scrapes, flaking streaks, and loss of coating to the base metal. In the areas where loss of protective coatings occurred, base metal had only minor localized corrosion with no notable signs of significant material loss. The conditions are expected conditions based on age and service condition of the component.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The degradation is not a defective condition with respect to the component function of pressure boundary or structural integrity. Therefore, additional examinations are not warranted.

(3) A description of the necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

100% of the PSC exterior surface was examined in the air/water interface area. Based on the above evaluation, no additional examinations are warranted.



APPENDIX IWE

Report No: CISI-057

Component: Pressure Suppression Chamber
Exterior Surface

Condition/Indication: Indications were noted during the VT-1 examination of the Pressure Suppression Chamber (PSC) Exterior Surface air/water interface region.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A).

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coating discontinuities, including minor scratches, scrapes, flaking streaks, and loss of coating to the base metal. In the areas where loss of protective coatings occurred, base metal had only minor localized corrosion with no notable signs of significant material loss. The conditions are expected conditions based on age and service condition of the component.

(2) Evaluation of each area, and the result of the evaluation:

The conditions noted do not impact the structural integrity or leak tightness of the PSC, and are expected conditions based on the age and service condition of the component. The primary reason this area is being inspected is due to the conditions which could potentially affect the interior surface. There is no accelerated degradation exhibited by this component. Therefore, no degradation exists which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coating discontinuities, including minor scratches, scrapes, flaking streaks, and loss of coating to the base metal. In the areas where loss of protective coatings occurred, base metal had only minor localized corrosion with no notable signs of significant material loss. The conditions are expected conditions based on age and service condition of the component.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The degradation is not a defective condition with respect to the component function of pressure boundary or structural integrity. Therefore, additional examinations are not warranted.

(3) A description of the necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

100% of the PSC exterior surface was examined in the air/water interface area. Based on the above evaluation, no additional examinations are warranted.



APPENDIX IWE

Report No: CISI-066

Component: CRD Removal Hatch
2-X-6

Condition/Indication: Indications noted during the VT-1 examination of CRD Removal Hatch flange mating surface. This examination was performed to satisfy the examination of bolted connections per Table IWE-2500 Category E-G.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

- (1) **Description of the type and estimated extent of degradation, and the conditions that led to the degradation:**

A nick was identified at the inboard edge of the outer o-ring groove. The indication appears to be an old indication resulting from mechanical damage during outages when the hatch is used for CRD changeout. The indication (1/8 inch wide by 1/16 inch deep) does not exhibit any sharp edges or characteristics of a crack-like indication.

- (2) **Evaluation of each area, and the result of the evaluation:**

The indication is not a defect with respect to the functional capability of the hatch. This indication is not deep enough to affect the structural integrity or sealing capability of the primary containment vessel. Successful as-found and as-left leak test results demonstrate the indication does not impact the leak tightness of the hatch. Therefore, there are no adverse conditions which may be present in inaccessible areas.

- (3) **Description of necessary corrective actions:**

No corrective actions are warranted.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

- (1) **Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:**

A nick was identified at the inboard edge of the outer o-ring groove. The indication appears to be an old indication resulting from mechanical damage during outages when the hatch is used for CRD changeout. The indication (1/8 inch wide by 1/16 inch deep) does not exhibit any sharp edges or characteristics of a crack-like indication.

- (2) **The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:**

The indication is not a defect with respect to the functional capability of the hatch. This indication is not deep enough to affect the structural integrity or sealing capability of the primary containment vessel. Successful as-found and as-left leak test results demonstrate the indication does not impact the leak tightness of the hatch. Therefore, there are no adverse conditions which may be present in inaccessible areas.

- (3) **A description of the necessary corrective actions:**

No corrective actions are warranted.

- (4) **The number and type of additional examinations to ensure detection of similar degradation in similar components:**

No additional examinations are warranted.



APPENDIX IWE

Report No: CISI-070

Component: Personnel Airlock Door Handle
Seals
2-X-2

Condition/Indication: Indications noted during the VT-3 examination of the Personnel Airlock Door Handle Seals. This examination was performed to satisfy the examination of bolted connections per Table IWE-2500-1 Category E-D.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The examination revealed that the o-rings that were being replaced had noteworthy degradation. The handle o-rings were broken, cracked, dried out, and worn. The conditions are primarily due to the age of the seals and mechanical wear from use.

(2) Evaluation of each area, and the result of the evaluation:

The indication is not a defective condition with respect to the acceptance criteria of IWE-3513.1. Successful as-found test results demonstrate the indication does not impact the leak tightness of the hatch. Therefore, there are no adverse conditions which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are warranted.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

The examination revealed that the o-rings that were being replaced had noteworthy degradation. The handle o-rings were broken, cracked, dried out, and worn. The conditions are primarily due to the age of the seals and mechanical wear from use.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The indication is not a defective condition with respect to the acceptance criteria of IWE-3513.1. Successful as-found test results demonstrate the indication does not impact the leak tightness of the hatch. Therefore, there are no adverse conditions which may be present in inaccessible areas.

(3) A description of the necessary corrective actions:

No corrective actions are warranted.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

No additional examinations are warranted.



APPENDIX IWE

Report No: CISI-081

Component: Shear Lug Access Hatch
O-Rings 2-7A/B, 2-8A/B

Condition/Indication: Indications noted during the VT-3 examination of the Personnel Airlock Door Handle Seals. This examination was performed to satisfy the examination of bolted connections per Table IWE-2500-1. Category E-D.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The examination revealed that the o-rings that were being replaced as a scheduled maintenance activity had noteworthy degradation. The o-rings were found to be hard and brittle. The conditions are primarily due to the age of the seals. The planned replacement of these o-rings in conjunction with local leak rate testing assures the leak tightness of the affected hatches. The remaining hatches had satisfactory as-found leak test results providing assurance that the degradation found on the examined components is not present in other areas.

(2) Evaluation of each area, and the result of the evaluation:

The remaining hatches had satisfactory as-found leak test results providing assurance that the degradation found on the examined components is not present in other areas. Therefore, there are no adverse conditions which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are warranted.

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

(1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:

The examination revealed that the o-rings that were being replaced as a scheduled maintenance activity had noteworthy degradation. The o-rings were found to be hard and brittle. The conditions are primarily due to the age of the seals. The planned replacement of these o-rings in conjunction with local leak rate testing assures the leak tightness of the affected hatches. The remaining hatches had satisfactory as-found leak test results providing assurance that the degradation found on the examined components is not present in other areas.

(2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:

The remaining hatches had satisfactory as-found leak test results providing assurance that the degradation found on the examined components is not present in other areas. Therefore, there are no adverse conditions which may be present in inaccessible areas.

(3) A description of the necessary corrective actions:

No corrective actions are warranted.

(4) The number and type of additional examinations to ensure detection of similar degradation in similar components:

No additional examinations are warranted.



APPENDIX IWE

Report No: CISI-085

Component: Pressure Suppression Chamber
Exterior Surface 2-B-1, 2, 6, 12

Condition/Indication: Indications were noted during the VT-1 examination of the Pressure Suppression Chamber (PSC) Exterior Surface air/water interface region outer radius of Bays 1, 2, 6, and 12.

EVALUATION OF INACCESSIBLE AREAS 10CFR50.55a(b)(2)(x)(A)

(1) Description of the type and estimated extent of degradation, and the conditions that led to the degradation:

The indications consist of numerous areas of coatings distress, ranging from minor scratches, scrapes, flaking, stains, and loss of protective coatings to the base metal. The areas where the loss of coatings to the base metal were observed had minor localized corrosion with no significant signs of material loss. The noted conditions do not impact the structural integrity or leak tightness of the pressure suppression chamber. The indications noted are expected conditions based on the age and service condition of the pressure suppression chamber.

(2) Evaluation of each area, and the result of the evaluation:

The conditions noted do not impact the structural integrity or leak tightness of the PSC and are expected conditions based on the age and service condition of the component. VT-1 examinations of Bays 1, 2, 6, and 12 did not show any signs of accelerated degradation on the outside radius. Therefore, no degradation exists which may be present in inaccessible areas.

(3) Description of necessary corrective actions:

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.



APPENDIX IWE

Report No: CISI-085 (Continued)

Component: Pressure Suppression Chamber
Exterior Surface 2-B-1, 2, 6, 12

ADDITIONAL EXAMINATIONS 10CFR 50.55a(b)(2)(x)(D)

- (1) Description of each flaw or area, including the extent of degradation, and the conditions that led to the degradation:**

The indications consist of numerous areas of coatings distress, ranging from minor scratches, scrapes, flaking, stains, and loss of protective coatings to the base metal. The areas where the loss of coatings to the base metal were observed had minor localized corrosion with no significant signs of material loss. The noted conditions do not impact the structural integrity or leak tightness of the pressure suppression chamber. The indications noted are expected conditions based on the age and service condition of the pressure suppression chamber.

- (2) The acceptability of each flaw or area, and the need for additional examinations to verify that similar degradation does not exist in similar components:**

The degradation is not a defective condition with respect to the component function of pressure boundary or structural integrity. Therefore, additional examinations are not warranted.

- (3) A description of the necessary corrective actions:**

No corrective actions are required. The coatings may be repaired as part of the routine coatings maintenance program.

- (4) The number and type of additional examinations to ensure detection of similar degradation in similar components:**

100% of the PSC exterior surface was examined in the air/water interface area. Based on the above evaluation, no additional examinations are warranted.



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

ATTACHMENT 3

SUPPLEMENTAL SUBMITTAL (CORRECTED INFORMATION - BFN UNIT 3 ISI SECOND INSPECTION INTERVAL (1ST PERIOD UNIT 3 CYCLE 8 OUTAGE CONDUCTED IN SEPT./OCT. 1998)

Component 3-FCV-1-037 Main Steam System Valve received a VT-3 valve internal examination, Code Category B-M-2, Item # B12.50 for code credit (89E-02) documented on Report # R-223. This was documented in the Unit 3 Cycle 8 Refueling Outage NIS-1 90 Day Summary Report and submitted to the NRC on January 12, 1999, reference RIMS # R08 990112 750. This examination was actually a Preservice Examination performed to Work Order # 98-002608-003. The Unit 3 Cycle 8 Refueling Outage NIS-1 90 Day Summary Report has been revised to delete the aforementioned component from the NIS-1 Report as an Inservice Inspection examination for code credit (89E-02) and is now listed as a Preservice Examination (P07-02). See attached revised report. This correction is being submitted to the NRC with the Unit 2 Cycle 10 Refueling Outage NIS-1 90 Day Report.



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 89E-02

UNIT: THREE CYCLE: 8

COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
CSS	FCV-75-54-BC	ISI-0331-C 01	B-G-2	B7.70	VT-1		19980922	R010	P	
CSS	TCS-3-406	ISI-0331-C 01	B-J	B9.11	PT		19980921	R012	P	
CSS	TCS-3-406	ISI-0331-C 01	B-J	B9.11	UT	BF-75	19980926	R047	P	
CSS	TCS-3-417	ISI-0331-C 01	B-F	B5.10	PT		19980922	R015	P	
CSS	TCS-3-417	ISI-0331-C 01	B-F	B5.10	UT	BF-106	19980924	R179	P	
CSS	TCS-3-417	ISI-0331-C 01	B-F	B5.10	UT	BF-65	19980924	R179	P	
CSS	TCS-3-426	ISI-0331-C 01	B-J	B9.11	PT		19980921	R011	P	
CSS	TCS-3-426	ISI-0331-C 01	B-J	B9.11	UT	BF-75	19980924	R046	P	
CSS	TSCS-3-418	ISI-0331-C 01	B-J	B9.11	PT		19980922	R014	P	
CSS	TSCS-3-418	ISI-0331-C 01	B-J	B9.11	UT	BF-05	19980924	R180	P	
CSS	TSCS-3-418	ISI-0331-C 01	B-J	B9.11	UT	BF-46	19980924	R180	P	
FWS	3-47B415-36-IA	ISI-0336-C 01	B-K	B10.10	MT		19981004	R109	P	1ST ADDITIONAL SAMPLE FOR NOI U3C8-011
FWS	3-47B415-38	ISI-0336-C 01	F-A	F1.10C	VT-3		19980929	R059	P	
FWS	3-47B415-38-IA	ISI-0336-C 01	B-K-1	B10.10	MT		19980928	R042	P	REPAIRED AND REEXAMINED UNDER R198
FWS	3-47B415-42-IA	ISI-0336-C 01	B-K-1	B10.10	MT		19981007	R133	P	1ST ADDITIONAL SAMPLE FOR U3C8-011
FWS	3-47B415-47-IA	ISI-0336-C 01	B-K-1	B10.10	MT		19981004	R108	P	1ST ADDITIONAL SAMPLE FOR NOI U3C8-011
FWS	3-47B415-58-IA	ISI-0336-C 01	B-K-1	B10.10	MT		19981007	R134	P	1ST ADDITIONAL SAMPLE FOR U3C8-011
FWS	3-47B415-60-IA	ISI-0336-C 01	B-K-1	B10.10	MT		19980928	R055	P	
MSS	3-47B400-081-IA	ISI-0355-C 01	C-C	C3.20	MT		19980930	R049	P	
MSS	3-47B400-083	ISI-0338-C 01	F-A	F1.10C	VT-3		19981006	R131	P	
MSS	3-47B400-083-IA	ISI-0338-C 01	B-K-1	B10.10	MT		19980929	R056	P	
MSS	3-47B400-114-IA	ISI-0338-C 02	B-K-1	B10.10	MT		19980930	R048	P	



OWNER: TENNESSEE VALLEY AUTHORITY
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 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 89E-02

UNIT: THREE CYCLE: 8

COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
MSS	DMS-3-29	ISI-0354-C 01	C-F-2	C5.81	MT		19981003	R095	P	
MSS	FCV-1-027-BC	ISI-0329-C 02	B-G-2	B7.70	VT-1		19980928	R058	P	
MSS	HPAS-3-H-03	ISI-0355-C 02	F-A	F1.20C	VT-3		19981002	R096	P	
MSS	HPAS-3-H-10	ISI-0355-C 02	F-A	F1.20C	VT-3		19981002	R120	P	
MSS	KMS-3-114	ISI-0329-C 02	B-J	B9.11	MT		19981006	R136	P	
MSS	KMS-3-114	ISI-0329-C 02	B-J	B9.11	UT	BF-104	19981007	R146	P	
MSS	MS-3-H-09	ISI-0355-C 01	F-A	F1.20C	VT-3		19981011	R200	P	PART OF ADDITIONAL SAMPLE FOR NOI-26
MSS	MS-3-H-12	ISI-0355-C 01	F-A	F1.20C	VT-3		19981002	R119	P	
MSS	MS-3-H-13	ISI-0355-C 01	F-A	F1.20C	VT-3		19981011	R150	P	REINSP. REPORT R201 SETTING PASSED
MSS	MS-3-H-13-IA	ISI-0355-C 01	C-C	C3.20	MT		19981002	R088	P	
MSS	MS-3-H-15-IA	ISI-0355-C 01	C-C	C3.20	MT		19981002	R089	P	
MSS	MS-3-H-16-IA	ISI-0355-C 01	C-C	C3.20	MT		19981002	R117	P	
MSS	MS-3-H-17	ISI-0355-C 01	F-A	F1.20C	VT-3		19981002	R097	P	COMPONENT PASSED REINSPECTION ON R199
MSS	PCV1-3-004-PBC	ISI-0313-B 01	B-G-2	B7.50	VT-1		19980925	R025	P	
MSS	PCV1-3-018	ISI-0313-B 01	B-G-2	B7.70	VT-3		19981005	R138	P	VALVE SERIAL NO. 581030 & REPORT R219
MSS	PCV1-3-018	ISI-0313-B 01	B-M-2	B12.50	VT-3		19981005	R138	P	VALVE SERIAL NO. 581030 & REPORT R220
MSS	PCV1-3-018-PBC	ISI-0313-B 01	B-G-2	B7.50	VT-1		19980925	R026	P	
MSS	PCV1-3-018-VBC	ISI-0313-B 01	B-G-2	B7.70	VT-1		19980925	R027	P	
MSS	PCV1-3-034-PBC	ISI-0313-B 01	B-G-2	B7.50	VT-1		19980925	R028	P	
MSS	PCV1-3-034-VBC	ISI-0313-B 01	B-G-2	B7.70	VT-1		19980925	R029	P	
MSS	PCV1-3-042-PBC	ISI-0313-B 01	B-G-2	B7.50	VT-1		19980925	R030	P	
MSS	PCV1-3-042-VBC	ISI-0313-B 01	B-G-2	B7.70	VT-1		19980925	R031	P	



OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 89E-02

UNIT: THREE CYCLE: 8

COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	3-47B465-497	ISI-0337-C 02	F-A	F1.40D	VT-3		19980924	R022	P	
RECIR	3-47B465-498	ISI-0337-C 02	F-A	F1.40D	VT-3		19980924	R020	P	
RECIR	3-47B465-499	ISI-0337-C 02	F-A	F1.40D	VT-3		19980925	R035	P	
RECIR	3-47B465-500	ISI-0337-C 02	F-A	F1.10C	VT-3		19980924	R021	P	
RECIR	3-47B465-501	ISI-0337-C 02	F-A	F1.10C	VT-3		19980925	R032	P	
RECIR	3-47B465-502	ISI-0337-C 02	F-A	F1.10C	VT-3		19980925	R033	P	
RECIR	3-47B465-503	ISI-0337-C 02	F-A	F1.10D	VT-3		19980924	R023	P	
RECIR	GR-3-62	ISI-0328-C 02	B-J	B9.11	PT		19980924	R018	P	
RECIR	GR-3-62	ISI-0328-C 02	B-J	B9.11	UT	BF-57	19981006	R171	P	
RHRS	DRHR-3-02	ISI-0393-C 10	C-F-1	C5.11	PT		19980917	R009	P	
RHRS	DRHR-3-02	ISI-0393-C 10	C-F-1	C5.11	UT	BF-102	19980921	R178	P	
RHRS	DRHR-3-06	ISI-0330-C 01	B-J	B9.11	UT	BF-57	19981006	R155	P	
RHRS	DRHR-3-11	ISI-0393-C 11	C-F-1	C5.11	PT		19980917	R013	P	
RHRS	DRHR-3-11	ISI-0393-C 11	C-F-1	C5.11	UT	BF-102	19980919	R177	P	
RHRS	DRHR-3-14	ISI-0330-C 01	B-J	B9.11	PT		19981004	R106	P	
RHRS	DRHR-3-14	ISI-0330-C 01	B-J	B9.11	UT	BF-57	19981004	R116	P	
RHRS	DSRHR-3-05	ISI-0330-C 01	B-J	B9.11	PT		19981005	R129	P	SUBSTITUTE FOR DRHR-3-06
RHRS	DSRHR-3-05	ISI-0330-C 01	B-J	B9.11	UT	BF-57	19981004	R115	P	
RHRS	DSRHR-3-08	ISI-0330-C 01	B-J	B9.31	PT		19980924	R019	P	
RHRS	DSRHR-3-08	ISI-0330-C 01	B-J	B9.31	UT	BF-87	19980925	R024	P	
RHRS	FCV74-54-BC	ISI-0330-C 01	B-G-2	B7.70	VT-1		19981004	R123	P	VT FOR LEAKAGE
RHRS	TRHR-3-191	ISI-0330-C 01	B-J	B9.11	PT		19981005	R118	P	



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PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 89E-02

UNIT: THREE CYCLE: 8

COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	TRHR-3-191	ISI-0330-C 01	B-J	B9.11	UT	BF-99	19981007	R151	P	
RPV	3-SI-3.3.1.A		B-P	B15.10	VT-2		19981013	R226	P	
RPV	3-SI-3.3.1.A		B-P	B15.50	VT-2		19981013	R226	P	
RPV	3-SI-3.3.1.A		B-P	B15.60	VT-2		19981013	R226	P	
RPV	3-SI-3.3.1.A		B-P	B15.70	VT-2		19981013	R226	P	
RPV	3-SI-3.3.1.C		C-H	C7.30	VT-2		19980622	R224	P	
RPV	3-SI-3.3.1.C		C-H	C7.30	VT-2		19981015	R227	P	
RPV	3-SI-3.3.1.C		C-H	C7.70	VT-2		19980622	R224	P	
RPV	3-SI-3.3.1.C		C-H	C7.70	VT-2		19981015	R227	P	
RPV	3-SI-3.3.11		C-H	C7.30	VT-2		19981008	R235	P	
RPV	3-SI-3.3.11		C-H	C7.70	VT-2		19981008	R235	P	
RPV	3-SI-3.3.2.A		C-H	C7.30	VT-2		19981006	R225	P	
RPV	3-SI-3.3.2.A		C-H	C7.70	VT-2		19981006	R225	P	
RPV	3-SI-3.3.2.B		C-H	C7.30	VT-2		19981006	R228	P	
RPV	3-SI-3.3.2.B		C-H	C7.70	VT-2		19981006	R228	P	
RPV	3-SI-3.3.4		C-H	C7.30	VT-2		19980926	R231	P	
RPV	3-SI-3.3.4		C-H	C7.50	VT-2		19980926	R231	P	
RPV	3-SI-3.3.4		C-H	C7.70	VT-2		19980926	R231	P	
RPV	3-SI-3.3.5		C-H	C7.30	VT-2		19980923	R232	P	
RPV	3-SI-3.3.5		C-H	C7.70	VT-2		19980923	R232	P	
RPV	3-SI-3.3.7		C-H	C7.30	VT-2		19980909	R233	P	
RPV	3-SI-3.3.7		C-H	C7.70	VT-2		19980909	R233	P	



OWNER: TENNESSEE VALLEY AUTHORITY
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 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 89E-02

UNIT: THREE CYCLE: 8

COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	3-SI-3.3.8.B		C-H	C7.30	VT-2		19980922	R234	P	
RPV	3-SI-3.3.8.B		C-H	C7.70	VT-2		19980922	R234	P	
RPV	INTERIOR	ISI-0220-C 02	B-N-1	B13.10	VT		19981003	R218	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N1A	ISI-0328-C 01	B-D	B3.90	UT	BF-18	19981008	R205	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N1A-IR	ISI-0328-C 01	B-D	B3.100	UT	BF-18	19981008	R205A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N1A-SE	ISI-0328-C 01	B-F	B5.10	PT		19980926	R036	P	
RPV	N1A-SE	ISI-0328-C 01	B-F	B5.10	UT	BF-107	19981002	R153	P	
RPV	N1A-SE	ISI-0328-C 01	B-F	B5.10	UT	BF-90	19981002	R153	P	
RPV	N2B	ISI-0328-C 02	B-D	B3.90	UT	BF-18	19981008	R206	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N2B-IR	ISI-0328-C 02	B-D	B3.100	UT	BF-18	19981008	R206A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N2D	ISI-0328-C 02	B-D	B3.90	UT	BF-18	19981008	R207	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N2D-IR	ISI-0328-C 02	B-D	B3.100	UT	BF-18	19981008	R207A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N2F	ISI-0328-C 02	B-D	B3.90	UT	BF-18	19981008	R208	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N2F-IR	ISI-0328-C 02	B-D	B3.100	UT	BF-18	19981008	R208A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N3B	ISI-0329-C 02	B-D	B3.90	UT	BF-18	19981008	R209	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N3B-IR	ISI-0329-C 02	B-D	B3.100	UT	BF-18	19981008	R209A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N3B-SE	ISI-0329-C 02	B-J	B9.11	MT		19980927	R054	P	
RPV	N3B-SE	ISI-0329-C 02	B-J	B9.11	UT	BF-104	19981001	R085	P	
RPV	N4B	ISI-0327-C 01	B-D	B3.90	UT	BF-18	19981014	R211	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4B-IR	ISI-0327-C 01	B-D	B3.100	UT	BF-18	19981009	R211A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4B-SE	ISI-0327-C 01	B-J	B9.11	MT		19981004	R104	P	
RPV	N4B-SE	ISI-0327-C 01	B-J	B9.11	UT	BF-56	19981005	R142	P	



OWNER: TENNESSEE VALLEY AUTHORITY
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1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35609-2000

EXAM REQUIREMENT 89E-02

UNIT: THREE CYCLE: 8

COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	N4C	ISI-0327-C 01	B-D	B3.90	UT	BF-18	19981014	R212	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4C-IR	ISI-0327-C 01	B-D	B3.100	UT	BF-18	19981009	R212A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4C-SE	ISI-0327-C 01	B-J	B9.11	MT		19981004	R105	P	
RPV	N4C-SE	ISI-0327-C 01	B-J	B9.11	UT	BF-56	19981005	R141	P	
RPV	N5A	ISI-0331-C 01	B-D	B3.90	UT	BF-18	19981008	R216	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N5A-IR	ISI-0331-C 01	B-D	B3.100	UT	BF-18	19981008	R216A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N6A-3-1-BC	ISI-0295-A 01	B-G-2	B7.50	VT-1		19980926	R034	P	
RPV	N8A	ISI-0411-C 01	B-D	B3.90	UT	BF-18	19981009	R217	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N8A-IR	ISI-0411-C 01	B-D	B3.100	UT	BF-18	19981009	R217A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY



EXAM REQUIREMENTS

B01-02
 B02-02
 B03-02
 B04-02
 B06-02
 P07-02
 V01-02

**OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402**

**PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000**

UNIT: THREE CYCLE: 8 COMMERCIAL SERVICE DATE: MARCH 1, 1977

**CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED**

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
CRDS	RCRD-3-33	ISI-0332-C 02	B02-02	C	NU0313	UT	BF-60	19981003	R135	P	
CRDS	RCRD-3-33	ISI-0332-C 02	B02-02	C	NU0313	UT	BF-76	19981003	R135	P	
CRDS	RCRD-3-44	ISI-0332-C 02	B01-02	7112	V01-01	UT	BF-01	19981005	R147	P	
CRDS	RCRD-3-45	ISI-0332-C 02	B01-02	7112	V01-01	UT	BF-29	19981006	R149	P	
CRDS	RCRD-3-49	ISI-0332-C 02	B02-02	C	NU0313	UT	BF-29	19981006	R143	P	
CRDS	RCRD-3-50	ISI-0332-C 02	B02-02	C	NU0313	UT	BF-29	19981006	R144	P	
CRDS	RCRD-3-52	ISI-0332-C 02	B02-02	C	NU0313	UT	BF-29	19981006	R145	P	
CRDS	RCRDS-3-03	ISI-0332-C 02	B01-02	7112	V01-01	UT	BF-35	19981004	R148	P	
CSS	DCS-3-04	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980922	R121	P	
CSS	DCS-3-05	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980923	R187	P	
CSS	DCS-3-13	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980924	R040	P	
CSS	DCS-3-14	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980923	R188	P	
CSS	DSCS-3-01	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980923	R191	P	
CSS	DSCS-3-02	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980923	R190	P	
CSS	DSCS-3-07	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980923	R189	P	
CSS	DSCS-3-08	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980923	R039	P	
CSS	DSCS-3-09	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-79	19980924	R192	P	
CSS	TCS-3-405	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-75	19980926	R066	P	
CSS	TCS-3-406	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-75	19980926	R047	P	
CSS	TCS-3-410	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-75	19980930	R069	P	
CSS	TCS-3-421	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-75	19980927	R045	P	
CSS	TCS-3-422	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-75	19980927	R067	P	



EXAM REQUIREMENTS

B01-02
B02-02
B03-02
B04-02
B06-02
P07-02
V01-02

**OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402**

**PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35609-2000**

UNIT: THREE CYCLE: 8 COMMERCIAL SERVICE DATE: MARCH 1, 1977

**CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED**

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
CSS	TCS-3-426	ISI-0331-C 01	B02-02	C	NU0313	UT	BF-75	19980924	R046	P	
CSS	TSCS-3-408	ISI-0331-C 01	B04-02	B-J	TS46G	MT		19980927	R017	P	
CSS	TSCS-3-408	ISI-0331-C 01	B04-02	B-J	TS46G	UT	BF-75	19981001	R107	P	
CSS	TSCS-3-423	ISI-0331-C 01	B04-02	B-J	TS46G	MT		19980923	R016	P	
CSS	TSCS-3-423	ISI-0331-C 01	B04-02	B-J	TS46G	UT	BF-75	19981001	R111	P	
FWS	GFW-3-12	ISI-0327-C 01	B04-02	B-J	TS46G	MT		19980927	R038	P	
FWS	GFW-3-12	ISI-0327-C 01	B04-02	B-J	TS46G	UT	BF-75	19980930	R080	P	
FWS	GFW-3-26	ISI-0327-C 01	B04-02	B-J	TS46G	MT		19980927	R037	P	
FWS	GFW-3-26	ISI-0327-C 01	B04-02	B-J	TS46G	UT	BF-75	19980930	R079	P	
FWS	KFW-3-39	ISI-0327-C 01	B04-02	B-J	TS46G	MT		19980928	R057	P	
FWS	KFW-3-39	ISI-0327-C 01	B04-02	B-J	TS46G	UT	BF-75	19980930	R078	P	
HPCIS	3-47B455-626	CHM-2413-C 02	P07-02	F-A	F1.20C	VT-1		19981003	R103	P	
HPCIS	HPCI-3-019-009	CHM-2407-C 02	P07-02	C-F	C5.51	MT		19981003	R091	P	
HPCIS	HPCI-3-019-009	CHM-2407-C 02	P07-02	C-F	C5.51	UT	BF-42	19981003	R098	P	
HPCIS	HPCI-3-019-010	CHM-2407-C 02	P07-02	C-F-2	C5.51	MT		19980921	R003	P	
HPCIS	HPCI-3-019-010	CHM-2407-C 02	P07-02	C-F-2	C5.51	UT	BF-42	19980824	R004	P	
HPCIS	HPCI-3-019-011	CHM-2407-C 02	P07-02	C-F	C5.51	MT		19981003	R092	P	
HPCIS	HPCI-3-019-011	CHM-2407-C 02	P07-02	C-F	C5.51	UT	BF-42	19981004	R099	P	
HPCIS	THPCI-3-070	ISI-0333-C 01	B04-02	B-J	TS46G	MT		19980929	R052	P	
HPCIS	THPCI-3-070	ISI-0333-C 01	B04-02	B-J	TS46G	UT	BF-05	19981005	R053	P	
MSS	FCV-1-037	ISI-0329-C 02	P07-02	B-M-2	B12.50	VT-3		19980508	R223	P	
MSS	GMS-3-15	ISI-0329-C 02	B04-02	B-J	TS46G	MT		19980927	R050	P	



EXAM REQUIREMENTS

B01-02
 B02-02
 B03-02
 B04-02
 B06-02
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 V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
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 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: THREE CYCLE: 8 COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
MSS	GMS-3-15	ISI-0329-C 02	B04-02	B-J	TS46G	UT	BF-104	19981006	R132	P	
MSS	GMS-3-32	ISI-0329-C 01	B04-02	B-J	TS46G	MT		19980929	R051	P	
MSS	GMS-3-32	ISI-0329-C 01	B04-02	B-J	TS46G	UT	BF-104	19981004	R110	P	
MSS	HPAS-3-H-02	ISI-0355-C 02	P07-02	F-A	F1.20C	VT-3		19981002	R127	P	
MSS	HPAS-3-H-07	ISI-0355-C 02	P07-02	F-A	F1.20C	VT-3		19981002	R128	P	
MSS	PCV1-3-018-PBC	ISI-0313-B 01	P07-02	B-G-2	B7.50	VT-3		19981005	R139	P	
MSS	PCV1-3-179-BC	ISI-0313-B 01	P07-02	B-G-2	B7.70	VT-3		19981005	R140	P	
MSS	PCV1-3-179-PBC	ISI-0313-B 01	P07-02	B-G-2	B7.50	VT-1		19981003	R126	P	
MSS	PCV1-3-179-VBC	ISI-0313-B 01	P07-02	B-G-2	B7.50	VT-1		19981003	R125	P	
RECIR	3-47B465-2048	ISI-0337-C 01	V01-02	F-A	F1.10C	VT-1		19981004	R122	P	
RECIR	GR-3-01	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980929	R169	P	
RECIR	GR-3-02	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980929	R170	P	
RECIR	GR-3-04	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-02	19980929	R082	P	
RECIR	GR-3-07	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-02	19980930	R081	P	
RECIR	GR-3-28	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19980927	R167	P	
RECIR	GR-3-29	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19980926	R176	P	
RECIR	GR-3-30	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-02	19980925	R071	P	
RECIR	GR-3-33	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-02	19980926	R073	P	
RECIR	GR-3-55	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980930	R159	P	
RECIR	GR-3-56	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980930	R157	P	
RECIR	GR-3-58	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980930	R175	P	
RECIR	GR-3-59(OL)	ISI-0328-C 02	B02-02	E	NU0313	UT	BF-83	19980929	R044	P	PER BFPER970437



EXAM REQUIREMENTS

B01-02
 B02-02
 B03-02
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 B06-02
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 V01-02

**OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402**

**PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000**

UNIT: THREE CYCLE: 8 COMMERCIAL SERVICE DATE: MARCH 1, 1977

**CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED**

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	GR-3-61	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19980929	R164	P	
RECIR	GR-3-62	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19981006	R171	P	
RECIR	GR-3-63	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19981004	R172	P	RESOLUTION DATA ON R041
RECIR	GR-3-63A	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-59	19980926	R074	P	
RECIR	GR-3-63B	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-59	19980926	R070	P	
RECIR	KR-3-01	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-59	19980929	R063	P	
RECIR	KR-3-02	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980930	R174	P	
RECIR	KR-3-04	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-59	19980930	R064	P	
RECIR	KR-3-23	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-59	19980928	R075	P	
RECIR	KR-3-24	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19980927	R165	P	
RECIR	KR-3-26	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-59	19980926	R072	P	
RECIR	KR-3-45	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980926	R161	P	
RECIR	KR-3-46	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19981002	R156	P	
RECIR	KR-3-47	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980930	R162	P	
RECIR	KR-3-48	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-57	19980929	R173	P	
RECIR	KR-3-49	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-59	19980929	R065	P	
RECIR	KR-3-50	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19980929	R163	P	
RECIR	KR-3-51	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19980927	R166	P	
RECIR	KR-3-52	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-57	19980927	R168	P	
RECIR	KR-3-53	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-59	19980927	R068	P	
RECIR	RWR-3-017-004	ISI-0328-C 01	P07-02	B-J	B9.20	PT		19981014	R204	P	
RECIR	RWR-3-017-005	ISI-0328-C 01	P07-02	B-J	B9.20	PT		19981014	R203	P	



EXAM REQUIREMENTS

B01-02
 B02-02
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 V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000

UNIT: THREE CYCLE: 8 COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RECIR	RWR-3-017-007	ISI-0328-C 01	P07-02	B-J	B9.20	PT		19981014	R204	P	
RECIR	RWR-3-017-009	ISI-0328-C 01	P07-02	B-J	B9.20	PT		19981014	R204	P	
RHRS	3-47B452-3179	ISI-0340-C 01	V01-02	N/A	N/A	VT-1		19981005	R137	P	
RHRS	DRHR-3-02	ISI-0393-C 10	B02-02	C	NU0313	UT	BF-102	19980921	R178	P	
RHRS	DRHR-3-03	ISI-0330-C 01	B02-02	D	NU0313	UT	BF-57	19981002	R094	P	LESS THAN 50% COVERAGE
RHRS	DRHR-3-03B	ISI-0330-C 01	B02-02	G	NU0313	VT-2		19981013	R202	P	
RHRS	DRHR-3-04	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19980930	R086	P	
RHRS	DRHR-3-05	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981002		P	
RHRS	DRHR-3-06	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981006	R155	P	
RHRS	DRHR-3-07	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981003	R158	P	
RHRS	DRHR-3-11	ISI-0393-C 11	B02-02	C	NU0313	UT	BF-102	19980919	R177	P	
RHRS	DRHR-3-12	ISI-0330-C 01	B02-02	D	NU0313	UT	BF-57	19981005	R130	P	LESS THAN 50% COVERAGE
RHRS	DRHR-3-13	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981003	R124	P	
RHRS	DRHR-3-13B	ISI-0330-C 01	B02-02	G	NU0313	VT-2		19981013	R202	P	
RHRS	DRHR-3-14	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981004	R116	P	
RHRS	DRHR-3-15	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981002	R186	P	
RHRS	DRHR-3-16	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981003	R182	P	
RHRS	DRHR-3-19	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-87	19981004	R113	P	
RHRS	DRHR-3-21	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-87	19981006	R197	P	
RHRS	DRHR-3-22	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-87	19981006	R196	P	
RHRS	DRHR-3-23	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-87	19981006	R195	P	
RHRS	DSRHR-3-01	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981005	R185	P	



EXAM REQUIREMENTS

B01-02
B02-02
B03-02
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V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: THREE CYCLE: 8

COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RHRS	DSRHR-3-02	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981005	R184	P	
RHRS	DSRHR-3-03	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981005	R160	P	
RHRS	DSRHR-3-04	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981001	R084	P	
RHRS	DSRHR-3-04A	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19980930	R083	P	
RHRS	DSRHR-3-05	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981004	R115	P	
RHRS	DSRHR-3-05A	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981004	R114	P	
RHRS	DSRHR-3-06	ISI-0330-C 01	B04-02	B-J	TS46G	PT		19981002	R093	P	
RHRS	DSRHR-3-06	ISI-0330-C 01	B04-02	B-J	TS46G	UT	BF-57	19981002	R183	P	
RHRS	DSRHR-3-06	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981002	R183	P	
RHRS	DSRHR-3-07	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-57	19981001	R181	P	
RHRS	DSRHR-3-08	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-87	19980925	R024	P	
RHRS	DSRHR-3-09	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-87	19981006	R194	P	
RHRS	DSRHR-3-10	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-87	19981006	R193	P	
RHRS	FCV74-68	ISI-0330-C 01	V01-02	B-G-2	B7.70	VT-1		19981001	R061	P	
RHRS	TRHR-3-191	ISI-0330-C 01	B02-02	C	NU0313	UT	BF-99	19981007	R151	P	
RPV	ACCHCOV-1	IVVI EXAM	B06-02	SIL462	71145	UT		19981007	R221E	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	ACCHCOV-2	IVVI EXAM	B06-02	SIL462	71145	UT		19981007	R221F	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CORESHR-3-H-1	IVVI EXAM	B06-02	SIL572	71144	UT		19981008	R221	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CORESHR-3-H-2	IVVI EXAM	B06-02	SIL572	71144	UT		19981007	R221A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CORESHR-3-H-3	IVVI EXAM	B06-02	SIL572	71144	UT		19981007	R221B	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CORESHR-3-H-4	IVVI EXAM	B06-02	SIL572	71144	UT		19981007	R221C	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CORESHR-3-H-5	IVVI EXAM	B06-02	SIL572	71144	UT		19981007	R221D	P	EXAM PERFORMED BY GE NUCLEAR ENERGY



EXAM REQUIREMENTS

B01-02
 B02-02
 B03-02
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 B06-02
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 V01-02

**OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402**

**PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000**

UNIT: THREE CYCLE: 8 COMMERCIAL SERVICE DATE: MARCH 1, 1977

**CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED**

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	CS-DNCMRA-4A	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRA-4B	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRA-4C	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRA-4D	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRA-5	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRA-6	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRA-7	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRA-8A	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRA-8B	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-4A	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-4B	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-4C	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-4D	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-5	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-6	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-7	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-8A	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRB-8B	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRC-4A	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRC-4B	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRD-4A	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRD-4B	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY



EXAM REQUIREMENTS

B01-02
B02-02
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V01-02

OWNER: TENNESSEE VALLEY AUTHORITY
NUCLEAR POWER GROUP
1101 MARKET STREET
CHATTANOOGA, TENNESSEE 37402

PLANT: BROWNS FERRY NUCLEAR PLANT
P.O. BOX 2000
DECATUR, ALABAMA 35609-2000

UNIT: THREE CYCLE: 8

COMMERCIAL SERVICE DATE: MARCH 1, 1977

CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	CS-DNCMRD-4C	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRD-4D	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRD-5	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRD-6	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRD-7	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRD-8A	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	CS-DNCMRD-8B	IVVI EXAM	B03-02	SIL289	71141	UT		19981002	R222	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N1A-SE	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-107	19981002	R153	P	
RPV	N1A-SE	ISI-0328-C 01	B02-02	C	NU0313	UT	BF-90	19981002	R153	P	
RPV	N1B-SE	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-107	19981002	R154	P	
RPV	N1B-SE	ISI-0328-C 02	B02-02	C	NU0313	UT	BF-90	19981002	R154	P	
RPV	N4A(NOZ BORE)	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-93	19981009	R210	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4A-IR	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-18	19981009	R210	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4B(NOZ BORE)	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-93	19981009	R211A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4B-IR	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-18	19981009	R211A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4C(NOZ BORE)	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-93	19981009	R212A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4C-IR	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-18	19981009	R212A	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4D(NOZ BORE)	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-93	19981009	R213	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4D-IR	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-18	19981009	R213	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4E(NOZ BORE)	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-93	19981009	R214	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4E-IR	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-18	19981009	R214	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RPV	N4F(NOZ BORE)	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-93	19981009	R215	P	EXAM PERFORMED BY GE NUCLEAR ENERGY



EXAM REQUIREMENTS

B01-02
 B02-02
 B03-02
 B04-02
 B06-02
 P07-02
 V01-02

**OWNER: TENNESSEE VALLEY AUTHORITY
 NUCLEAR POWER GROUP
 1101 MARKET STREET
 CHATTANOOGA, TENNESSEE 37402**

**PLANT: BROWNS FERRY NUCLEAR PLANT
 P.O. BOX 2000
 DECATUR, ALABAMA 35609-2000**

UNIT: THREE CYCLE: 8 COMMERCIAL SERVICE DATE: MARCH 1, 1977

**CERTIFICATION OF AUTHORIZATION: NOT REQUIRED
 NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED**

System	Component Number	ISO Drawing	Exam Requirement	Category	Item Number	Exam Scheduled	Calibration Standard	Exam Date	Exam Report	Exam Results	Comments
RPV	N4F-IR	ISI-0327-C 01	B01-02	7112	NU0619	UT	BF-18	19981009	R215	P	EXAM PERFORMED BY GE NUCLEAR ENERGY
RWCU	3RWCU-09	ISI-0333-C 01	B02-02	C	NU0313	UT	BF-01	19980927	R076	P	
RWCU	RWCU-3-001-042	ISI-0332-C 01	P07-02	B-J	B9.11	PT		19980930	R062	P	SEE ALSO REPORT R077 (PASSED)
RWCU	RWCU-3-001-042	ISI-0332-C 01	P07-02	B-J	B9.11	UT	BF-31	19981001	R087	P	
RWCU	RWCU-3-001-043	ISI-0332-C 01	P07-02	B-J	B9.11	PT		19980927	R043	P	
RWCU	RWCU-3-001-043	ISI-0332-C 01	P07-02	B-J	B9.11	UT	BF-31	19981003	R101	P	
RWCU	RWCU-3-001-044	ISI-0332-C 01	P07-02	B-J	B9.11	PT		19981002	R090	P	SEE ALSO REPORT R060
RWCU	RWCU-3-001-044	ISI-0332-C 01	P07-02	B-J	B9.11	UT	BF-31	19981003	R102	P	
RWCU	RWCU-3-001-066	ISI-0332-C 01	P07-02	B-J	B9.11	PT		19981001	R060	P	
RWCU	RWCU-3-001-066	ISI-0332-C 01	P07-02	B-J	B9.11	UT	BF-31	19981004	R100	P	



OWNER: TENNESSEE VALLEY AUTHORITY PLANT: BROWNS FERRY NUCLEAR PLANT
OFFICE OF NUCLEAR POWER PO BOX 2000
1101 MARKET STREET DECATUR, ALABAMA 35609-2000
CHATTANOOGA, TENNESSEE 37402

UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

ATTACHMENT 4

SUPPLEMENTAL SUBMITTAL [TRANSFERRED INFORMATION -
BFN UNIT 2 ISI SECOND INSPECTION INTERVAL (1ST PERIOD)
UNIT 2 CYCLE 7 OUTAGE CONDUCTED IN OCT/NOV 1994
TRANSFERRED TO UNIT 2 SECOND INSPECTION INTERVAL (3RD
PERIOD) UNIT 2 CYCLE 10 CONDUCTED APR/MAY 1999]

<u>SYSTEM</u>	<u>COMPONENT ID</u>	<u>CATEGORY</u>	<u>ITEM NO.</u>
RHRWS	2-47B450R0023-IA	D-B	D2.20
RHRWS	2-47B450R0024-IA	D-B	D2.20
RHRWS	2-47B450R0027-IA	D-B	D2.20
RHRWS	2-47B450R0029-IA	D-B	D2.20
RHRWS	2-47B450R0030-IA	D-B	D2.20
RHRWS	2-47B450H0033-IA	D-B	D2.20
RHRWS	2-47B450H0034-IA	D-B	D2.20
RHRWS	2-47B450H0035-IA	D-B	D2.20
RHRWS	2-47B450R0035-IA	D-B	D2.20
RHRWS	2-47B450H0036-IA	D-B	D2.20
RHRWS	2-47B450H0040-IA	D-B	D2.40
RHRWS	2-47B450R0040-IA	D-B	D2.20
RHRWS	2-47B450H0041-IA	D-B	D2.40
RHRWS	2-47B450H0044-IA	D-B	D2.20
RHRWS	2-47B450H0049-IA	D-B	D2.20

The fifteen (15) components listed above were inspected during the Unit 2 Cycle 7 Refueling Outage (1st period) for ASME XI Code Credit under Code Category D-B Items # D2.20 and D2.40. These examinations were documented in the Unit 2 Cycle 7 Refueling Outage NIS-1 90 Day Summary Report and submitted to the NRC on February 13, 1995, reference RIMS # R08 950213 805.

The Unit 2 cycle 7 Refueling Outage was in the 1st Period of the Second Inspection Interval. During this outage the Browns Ferry Nuclear Plant Unit 2 Inservice Inspection Program Procedure 2-SI-4.6.G Revision 003, Section 8.1, "Examination Schedule Part III Class III Equivalent (IWD) Components", required these fifteen (15) components RHRWS System, Code Category D-B Item # D2.20 and D2.40 be inspected for Code Credit during the 1st Period of the Second Inspection Interval. Due to a reduction in the total number of inspections to be performed in the Second Inspection Interval sample of Code Category D-B Item # D2.20 and D2.40 in the RHRWS System, the 1st Period numbers had to be reduced.



OWNER: TENNESSEE VALLEY AUTHORITY OFFICE OF NUCLEAR POWER 1101 MARKET STREET CHATTANOOGA, TENNESSEE 37402	PLANT: BROWNS FERRY NUCLEAR PLANT PO BOX 2000 DECATUR, ALABAMA 35609-2000
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UNIT: TWO CERTIFICATE OF AUTHORIZATION: NOT REQUIRED.

COMMERCIAL SERVICE DATE: MARCH 1, 1975

NATIONAL BOARD NUMBER FOR UNIT: NOT REQUIRED.

(CONTINUED)

ATTACHMENT 4

SUPPLEMENTAL SUBMITTAL [TRANSFERRED INFORMATION -
BFN UNIT 2 ISI SECOND INSPECTION INTERVAL (1ST PERIOD)
UNIT 2 CYCLE 7 OUTAGE CONDUCTED IN OCT/NOV 1994
TRANSFERRED TO UNIT 2 SECOND INSPECTION INTERVAL (3RD
PERIOD) UNIT 2 CYCLE 10 CONDUCTED APR/MAY 1999]

This meant that the above fifteen (15) RHRSW System components had to be moved out of the 1st Period and placed into the 3rd Period in order to prevent the maximum code credited percentages in the 1st period from being exceeded in accordance with the ASME Section XI Code, No Addenda, Article IWB-2000, Paragraph IWB-2412, and Table IWB-2412-1. This change was made in the Browns Ferry Nuclear Plant Unit 2 Inservice Inspection Program Procedure 2-SI-4.6.G Revision 01.1, Section 8.1, "Examination Schedule Part III Class III Equivalent (IWD) Components."

The fifteen (15) RHRSW System components listed below were rescheduled and examined during the Unit 2 Cycle 10 Refueling Outage (3rd Period) for Code Credit for Code Category D-B Item # D2.20 and D2.40. These components will be documented in the Unit 2 cycle 10 Refueling Outage NIS-1 90 Day Summary Report and submitted to the NRC.

<u>SYSTEM</u>	<u>COMPONENT ID</u>	<u>CATEGORY</u>	<u>ITEM NO.</u>
RHRSW	2-47B450R0023-IA	D-B	D2.20
RHRSW	2-47B450R0024-IA	D-B	D2.20
RHRSW	2-47B450R0027-IA	D-B	D2.20
RHRSW	2-47B450R0029-IA	D-B	D2.20
RHRSW	2-47B450R0030-IA	D-B	D2.20
RHRSW	2-47B450H0033-IA	D-B	D2.20
RHRSW	2-47B450H0034-IA	D-B	D2.20
RHRSW	2-47B450H0035-IA	D-B	D2.20
RHRSW	2-47B450R0035-IA	D-B	D2.20
RHRSW	2-47B450H0036-IA	D-B	D2.20
RHRSW	2-47B450H0040-IA	D-B	D2.40
RHRSW	2-47B450R0040-IA	D-B	D2.20
RHRSW	2-47B450H0041-IA	D-B	D2.40
RHRSW	2-47B450H0044-IA	D-B	D2.20
RHRSW	2-47B450H0049-IA	D-B	D2.20



Revision 001

7/16/99

Total Examinations:

558

**TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR POWER PLANT - UNIT 2
EXAMS SCHEDULED FOR CYCLE 10**

SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
CRDS	2-ISV-085-612/3015		10	B15.70	B-P	R27-02	VT-1	N-VT-1				BVAL	
CRDS	RCRD-2-44	ISI-0272-C 01	10	NU0619	7126	B01-02	UT	N-UT-18	BF-01	04.00	0.337	PC,RED	PC,PIPE
CRDS	RCRD-2-45	ISI-0272-C 01	10	NU0619	7126	B01-02	UT	N-UT-18	BF-29	04.00	0.593	PC,PIPE	PC,RED
CRDS	RCRD-2-50	ISI-0272-C 01	10	NU0313	D	B02-02	UT	N-UT-64	BF-29	04.00	0.432	PE,EL	PC,VLV
CRDS	RCRDS-2-03	ISI-0272-C 01	10	NU0619	7126	B01-02	UT	N-UT-18	BF-35	08.00	0.593	PBCC,TR	PC,RED
CSS	2-47B458S0004	ISI-0280-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		12.00		HHS	
CSS	2-47B458S0013-IA	ISI-0280-C 01	10	B10.10	B-K-1	86E-02	MT	N-MT-6			0.750	PI,IWA	
CSS	2-CKV-75-0570D		10	B15.70	B-P	R27-02	VT-1	N-VT-1				BVAL	
CSS	DCS-2-04	ISI-0271-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-79	12.00	0.687	PP,PIPE	PC,PIPE
CSS	DCS-2-07	ISI-0271-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-79	12.00	0.687	PC,PIPE	PE,EL
CSS	DSCS-2-02	ISI-0271-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-79	12.00	0.687	PE,EL	PC,PIPE
CSS	DSCS-2-09	ISI-0271-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-79	12.00	0.687	PE,EL	PC,PIPE
CSS	HCV-75-27-BC	ISI-0271-C 01	10	B7.70	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
CSS	TCS-2-405	ISI-0271-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-51	12.00	0.687	PC,VLV	PE,EL
CSS	TCS-2-405	ISI-0271-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-75	12.00	0.687	PC,VLV	PE,EL
CSS	TCS-2-406	ISI-0271-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-75	12.00	0.687	PC,PIPE	PC,VLV
CSS	TCS-2-426	ISI-0271-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-75	12.00	0.687	PC,VLV	PE,EL
CSS	TSCS-2-408	ISI-0271-C 01	10	TS3432	B-J	B04-02	MT	N-MT-6		12.75	0.687	PC,PIPE	PE,EL
CSS	TSCS-2-408	ISI-0271-C 01	10	TS3432	B-J	B04-02	UT	N-UT-18	BF-75	12.75	0.687	PC,PIPE	PE,EL
EECWS	0-37B205S0059	ISI-0368-C 02	10	F1.30B	F-A	86E-02	VT-3	N-VT-1		18.00		HRH	
EECWS	1-47B451S0292	ISI-0368-C 10	10	F1.30B	F-A	86E-02	VT-3	N-VT-1		06.00		HRH	
EECWS	1-47B451S0301	ISI-0368-C 15	10	F1.30A	F-A	86E-02	VT-3	N-VT-1		06.00		HRS	
EECWS	1-47B451S0435	ISI-0368-C 10	10	F1.30B	F-A	86E-02	VT-3	N-VT-1		06.00		HRH	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPESA	COMPESB
EECWS	1-47B451S0443	ISI-0368-C 13	10	F1.30A	F-A	86E-02	VT-3	N-VT-1		18.00		HRS	
EECWS	2-47B451R0008	ISI-0368-C 06	10	F1.30A	F-A	86E-02	VT-3	N-VT-1		18.00		HRH	
EECWS	2-47B451R0031	ISI-0368-C 06	10	F1.30B	F-A	86E-02	VT-3	N-VT-1		18.00		HRS2	
EECWS	2-47B451R0033	ISI-0368-C 06	10	F1.30D	F-A	86E-02	VT-3	N-VT-1		18.00		HHS	HRH
EECWS	3-17B300S0141	ISI-0368-C 07	10	F1.30B	F-A	86E-02	VT-3	N-VT-1		18.00		HRH	
EECWS	3-17B300S2003	ISI-0368-C 07	10	F1.30B	F-A	86E-02	VT-3	N-VT-1		18.00		HRH	
EECWS	3-47B451H0006-IA	ISI-0368-C 09	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.188	PI,IWA	
EECWS	3-47B451H0020-IA	ISI-0368-C 08	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.322	PI,IWA	
EECWS	3-47B451H0023-IA	ISI-0368-C 08	10	D2.20	D-B	86E-02	VT-3	N-VT-1				PI,IWA	
EECWS	3-47B451R0001-IA	ISI-0368-C 09	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.875	PI,IWA	
EECWS	3-47B451R0003-IA	ISI-0368-C 09	10	D2.20	D-B	86E-02	VT-3	N-VT-1			1.500	PI,IWA	
EECWS	3-47B451R0005-IA	ISI-0368-C 09	10	D2.20	D-B	86E-02	VT-3	N-VT-1			1.500	PI,IWA	
EECWS	3-47B451R0015-IA	ISI-0368-C 08	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.750	PI,IWA	
EECWS	3-47B451R0020-IA	ISI-0368-C 08	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.500	PI,IWA	
EECWS	3-47B451R0023	ISI-0368-C 08	10	F1.30B	F-A	86E-02	VT-3	N-VT-1		18.00		HRS	
EECWS	3-47B451R0028-IA	ISI-0368-C 06	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.625	PI,IWA	
EECWS	3-47B451S0025-IA	ISI-0368-C 09	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.216	PI,IWA	
EECWS	3-47B451S0061-IA	ISI-0368-C 08	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.750	PI,IWA	
EECWS	3-47B451S0296-IA	ISI-0368-C 09	10	D2.20	D-B	86E-02	VT-3	N-VT-1			1.000	PI,IWA	
EECWS	3-47B451S0310-IA	ISI-0368-C 06	10	D2.20	D-B	86E-02	VT-3	N-VT-1			2.000	PI,IWA	
FPCS	2-47B454H0008	ISI-0133-C 01	10	F1.30A	F-A	86E-02	VT-3	N-VT-1		06.00		HRS2	
FPCS	2-47B454H0008-IA	ISI-0133-C 01	10	D3.20	D-C	86E-02	VT-3	N-VT-1			0.188	PI,IWA	
FPCS	2-47B454R0032	ISI-0133-C 01	10	F1.30B	F-A	86E-02	VT-3	N-VT-1		06.00		HRH	
FWS	2-47B415H0001-IA	ISI-0277-C 01	10	B10.30	B-K-1	86E-02	MT	N-MT-6			0.250	PI,IWA	
FWS	2-47B415H0007-IA	ISI-0277-C 01	10	B10.30	B-K-1	86E-02	MT	N-MT-6			2.000	PI,IWA	
FWS	2-47B415S0004	ISI-0277-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		12.00		HHS	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDI	NOMTHICK	COMPDESA	COMPDESB
FWS	2-47B415S0012	ISI-0277-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		12.00		HHS	
FWS	GFW-2-09	ISI-0269-C 01	10	TS3432	B-J	B04-02	MT	N-MT-6		12.75	0.844	PE,EL	PE,EL
FWS	GFW-2-09	ISI-0269-C 01	10	TS3432	B-J	B04-02	UT	N-UT-18	BF-93	12.75	0.844	PE,EL	PE,EL
FWS	GFW-2-32	ISI-0269-C 01	10	TS3432	B-J	B04-02	MT	N-MT-6		12.75	0.844	PC,TEE	PE,EL
FWS	GFW-2-32	ISI-0269-C 01	10	TS3432	B-J	B04-02	UT	N-UT-18	BF-93	12.75	0.844	PC,TEE	PE,EL
FWS	KFW-2-13	ISI-0269-C 01	10	TS3432	B-J	B04-02	MT	N-MT-6		12.75	0.844	PC,PIPE	PE,EL
FWS	KFW-2-13	ISI-0269-C 01	10	TS3432	B-J	B04-02	UT	N-UT-18	BF-93	12.75	0.844	PC,PIPE	PE,EL
HPCIS	2-47B455H0058	ISI-0130-C 01	10	F1.20A	F-A	86E-02	VT-3	N-VT-1		10.00		HRS	
HPCIS	2-47B455H0067	ISI-0130-C 02	10	F1.20A	F-A	86E-02	VT-3	N-VT-1		14.00		HRS	
HPCIS	2-47B455H0074	ISI-0130-C 03	10	F1.20A	F-A	86E-02	VT-3	N-VT-1		14.00		HRH	
HPCIS	2-47B455R0020	ISI-0130-C 01	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		10.00		HRH	
HPCIS	2-47B455S0009	ISI-0130-C 02	10	F1.20B	F-A	86E-02	VT-3	N-VT-1		14.00		HRS	
HPCIS	2-47B455S0010	ISI-0130-C 02	10	F1.20A	F-A	86E-02	VT-3	N-VT-1		14.00		HRS	
HPCIS	2-RTV-073-0225A		10	B15.70	B-P	R27-02	VT-1	N-VT-1				BVAL	
HPCIS	2-SI-3.3.09	N/A	10	C7.40	C-H	86E-02	VT-2	N-VT-4				E-LEAK	
HPCIS	2-SI-3.3.09	N/A	10	C7.60	C-H	86E-02	VT-2	N-VT-4				E-LEAK	
HPCIS	2-SI-3.3.09	N/A	10	C7.80	C-H	86E-02	VT-2	N-VT-4				E-LEAK	
HPCIS	HPCIH-2-1	ISI-0130-C 01	10	F1.40B	F-A	86E-02	VT-3	N-VT-1				HPMP	
HPCIS	HPCIH-2-2	ISI-0130-C 02	10	F1.40B	F-A	86E-02	VT-3	N-VT-1				HPMP	
HPCIS	HPCIH-2-3	ISI-0130-C 02	10	F1.40B	F-A	86E-02	VT-3	N-VT-1				HPMP	
HPCIS	THPCI-2-070A	ISI-0273-C 01	10	TS3432	B-J	B04-02	MT	N-MT-6		10.00	0.593	PC,PIPE	PC,PIPE
HPCIS	THPCI-2-070A	ISI-0273-C 01	10	TS3432	B-J	B04-02	UT	N-UT-18	BF-05	10.00	0.593	PC,PIPE	PC,PIPE
HPCIS	THPCI-2-071	ISI-0273-C 01	10	TS3432	B-J	B04-02	MT	N-MT-6		10.00	0.593	PC,PIPE	PC,VLV
HPCIS	THPCI-2-071	ISI-0273-C 01	10	TS3432	B-J	B04-02	UT	N-UT-18	BF-05	10.00	0.593	PC,PIPE	PC,VLV
HPCIS	THPCI-2-074	ISI-0273-C 01	10	B9.11	B-J	V01-02	MT	N-MT-6		10.00	0.593	PE,EL	PC,PIPE
HPCIS	THPCI-2-074	ISI-0273-C 01	10	B9.11	B-J	V01-02	UT	N-UT-18	BF-05	10.00	0.593	PE,EL	PC,PIPE



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
MSS	2-47B2401-35	ISI-0412-C 08	10	F1.20D	F-A	P10-02	VT-3	N-VT-1				HMS2	
MSS	2-47B400S0002	ISI-0079-C 01	10	F1.20B	F-A	86E-02	VT-3	N-VT-1		24.00		HRH	
MSS	2-47B400S0005	ISI-0279-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HRH	
MSS	2-47B400S0005-IA	ISI-0279-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			2.000	PI,IWA	
MSS	2-47B400S0006	ISI-0279-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HRH	
MSS	2-47B400S0006-IA	ISI-0279-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			2.000	PI,IWA	
MSS	2-47B400S0007	ISI-0279-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HRH	
MSS	2-47B400S0007-IA	ISI-0279-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			2.000	PI,IWA	
MSS	2-47B400S0008	ISI-0279-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HRH	
MSS	2-47B400S0008-IA	ISI-0279-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			2.000	PI,IWA	
MSS	2-47B400S0096-IA	ISI-0279-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9				PI,IWA	
MSS	2-47B400S0097	ISI-0279-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HHS	
MSS	2-47B400S0097-IA	ISI-0279-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			0.625	PI,IWA	
MSS	2-47B400S0103	ISI-0279-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HMS	
MSS	2-47B400S0103-IA	ISI-0279-C 02	10	B10.10	B-K-1	86E-02	MT	N-MT-6			0.500	PI,IWA	
MSS	2-47B400S0104	ISI-0279-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HHS	
MSS	2-47B400S0105	ISI-0279-C 02	10	F1.10D	F-A	86E-02	VT-3	N-VT-1		26.00		HHS	
MSS	2-47B400S0105	ISI-0279-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HHS	
MSS	2-47B400S0107	ISI-0279-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HHS	
MSS	2-47B400S0108	ISI-0279-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HHS	
MSS	2-47B400S0108-IA	ISI-0279-C 02	10	B10.10	B-K-1	86E-02	MT	N-MT-6			0.500	PI,IWA	
MSS	2-47B400S0109	ISI-0279-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HHS	
MSS	2-47B400S0110	ISI-0279-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		26.00		HHS	
MSS	2-47B400S0212	ISI-0279-C 01	10	F1.10D	F-A	86E-02	VT-3	N-VT-1		26.00		HMS2	
MSS	FCV-01-015	ISI-0222-C 01	10	B12.50	B-M-2	P10-02	VT-3	N-VT-1		26.00		AVALINT	
MSS	FCV-01-015	ISI-0222-C 01	10	B7.70	B-G-2	P10-02	VT-1	N-VT-1		26.00		AVALINT	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHID	NDEPROC	CALSTD	COMPDIA	NOMTHICK	COMPDESA	COMPDESB
MSS	FCV-01-015-BC	ISI-0222-C 01	10	B7.70	B-G-2	P10-02	VT-1	N-VT-1				BVAL	
MSS	FCV-01-038	ISI-0222-C 02	10	B12.50	B-M-2	86E-02	VT-3	N-VT-1		26.00		AVALINT	
MSS	FCV-01-038	ISI-0222-C 02	10	B12.50	B-M-2	P10-02	VT-3	N-VT-1		26.00		AVALINT	
MSS	GMS-2-15	ISI-0222-C 02	10	TS3432	B-J	B04-02	MT	N-MT-6		26.00	0.950	PE,EL	PBCC,T
MSS	GMS-2-15	ISI-0222-C 02	10	TS3432	B-J	B04-02	UT	N-UT-18	BF-104	26.00	0.950	PE,EL	PBCC,T
MSS	HPAS-2-H-05	ISI-0079-C 02	10	F1.20C	F-A	86E-02	VT-3	N-VT-1		06.00		HVS	
MSS	HPAS-2-H-06	ISI-0079-C 02	10	F1.20C	F-A	86E-02	VT-3	N-VT-1		06.00		HVS	
MSS	HPAS-2-H-11	ISI-0079-C 02	10	F1.20C	F-A	86E-02	VT-3	N-VT-1		06.00		HVS	
MSS	HPAS-2-H-11	ISI-0079-C 02	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		06.00		HVS	
MSS	KMS-2-024	ISI-0222-C 01	10	TS3432	B-J	B04-02	MT	N-MT-6		26.00	0.950	PE,EL	PC,HDR
MSS	KMS-2-024	ISI-0222-C 01	10	TS3432	B-J	B04-02	UT	N-UT-18	BF-104	26.00	0.950	PE,EL	PC,HDR
MSS	MS-2-H-14	ISI-0079-C 01	10	F1.20C	F-A	86E-02	VT-3	N-VT-1		24.00		HVS	
MSS	MSBC-2-08	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BFLG	
MSS	MSBC-2-09	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BFLG	
MSS	MSBC-2-10	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BFLG	
MSS	MSBC-2-11	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BFLG	
MSS	PCV1-2-004	ISI-0312-B 01	10	B12.50	B-M-2	86E-02	VT-3	N-VT-1				AVALINT	
MSS	PCV1-2-004	ISI-0312-B 01	10	B7.70	B-G-2	P10-02	VT-1	N-VT-1				AVALINT	
MSS	PCV1-2-004-PBC	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
MSS	PCV1-2-004-VBC	ISI-0312-B 01	10	B7.70	B-G-2	V01-02	VT-1	N-VT-1				BVAL	
MSS	PCV1-2-005-PBC	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
MSS	PCV1-2-018-PBC	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
MSS	PCV1-2-019-PBC	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
MSS	PCV1-2-022-PBC	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
MSS	PCV1-2-023	ISI-0312-B 01	10	B12.50	B-M-2	P10-02	VT-3	N-VT-1				AVALINT	
MSS	PCV1-2-023	ISI-0312-B 01	10	B7.70	B-G-2	P10-02	VT-1	N-VT-1				AVALINT	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDA	NOMTHICK	COMPESA	COMPESB
MSS	PCV1-2-023-PBC	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
MSS	PCV1-2-030	ISI-0312-B 01	10	B12.50	B-M-2	86E-02	VT-3	N-VT-1				AVALINT	
MSS	PCV1-2-030	ISI-0312-B 01	10	B7.70	B-G-2	86E-02	VT-1	N-VT-1				AVALINT	
MSS	PCV1-2-179-PBC	ISI-0312-B 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
RCICS	2-47B456-114-IE	ISI-0131-C 01	10	F1.40A	F-A	86E-02	VT-3	N-VT-1		06.00		HRS	
RCICS	2-47B456-116	ISI-0131-C 01	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		06.00		HHS2	
RCICS	2-47B456H0004	ISI-0131-C 01	10	F1.20A	F-A	86E-02	VT-3	N-VT-1		06.00		HRH	
RCICS	2-47B456H0025	ISI-0131-C 01	10	F1.20A	F-A	86E-02	VT-3	N-VT-1		06.00		HRS	
RCICS	2-47B456R0001	ISI-0131-C 01	10	F1.20B	F-A	86E-02	VT-3	N-VT-1		06.00		HRH	
RCICS	2-47B456R0007	ISI-0131-C 01	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		4.00		HMS	
RCICS	2-RTV-071.0001-BC		10	B15.70	B-P	R27-02	VT-1	N-VT-1				BVAL	
RCICS	2-SI-3.3.10	N/A	10	C7.40	C-H	86E-02	VT-2	N-VT-4				E-LEAK	
RCICS	2-SI-3.3.10	N/A	10	C7.60	C-H	86E-02	VT-2	N-VT-4				E-LEAK	
RCICS	2-SI-3.3.10	N/A	10	C7.80	C-H	86E-02	VT-2	N-VT-4				E-LEAK	
RCICS	RCIC-2-009-002	ISI-0129-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,PIPE	PC,VLV
RCICS	RCIC-2-009-003	ISI-0129-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,PIPE	PC,VLV
RCICS	RCIC-2-009-004	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,PIPE	PC,PIPE
RCICS	RCIC-2-009-009	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,PIPE	PC,PIPE
RCWS	1-47B446S0053-IA	ISI-0391-C 01	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.25	HGR	
RECIR	2-47B408S0040	ISI-0278-C 02	10	F1.10C	F-A	86E-02	VT-3	N-VT-1		28.00		HVS2	
RECIR	2-47B408S0040-IA	ISI-0278-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.500	PI,IWA	
RECIR	2-47B408S0041-IA	ISI-0278-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	
RECIR	2-47B408S0042	ISI-0278-C 02	10	F1.10C	F-A	A01-02	VT-3	N-VT-1		28.00		HHS	
RECIR	2-47B408S0042-IA	ISI-0278-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	
RECIR	2-47B408S0043	ISI-0278-C 02	10	F1.10C	F-A	86E-02	VT-3	N-VT-1		28.00		HCF	
RECIR	2-47B408S0043-IA	ISI-0278-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RECIR	2-47B408S0045-IA	ISI-0278-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.500	PI,IWA	
RECIR	2-47B408S0046	ISI-0278-C 02	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0047	ISI-0278-C 02	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0047-IA	ISI-0278-C 02	10	B10.20	B-K-1	86E-02	PT	N-PT-9			1.500	PI,IWA	
RECIR	2-47B408S0048	ISI-0278-C 02	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0053-IE	ISI-0278-C 02	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0054-IE	ISI-0278-C 02	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0055-IA	ISI-0278-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.500	PI,IWA	
RECIR	2-47B408S0057-IA	ISI-0278-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	
RECIR	2-47B408S0058	ISI-0278-C 01	10	F1.10D	F-A	86E-02	VT-3	N-VT-1		28.00		HHS	
RECIR	2-47B408S0058	ISI-0278-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		28.00		HHS	
RECIR	2-47B408S0058-IA	ISI-0278-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	
RECIR	2-47B408S0059	ISI-0278-C 01	10	F1.10C	F-A	A01-02	VT-3	N-VT-1		28.00		HCF	
RECIR	2-47B408S0059-IA	ISI-0278-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	
RECIR	2-47B408S0060	ISI-0278-C 01	10	F1.10C	F-A	A01-02	VT-3	N-VT-1		28.00		HVS2	
RECIR	2-47B408S0060-IA	ISI-0278-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.500	PI,IWA	
RECIR	2-47B408S0061	ISI-0278-C 01	10	F1.40D	F-A	86E-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0061	ISI-0278-C 01	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0062	ISI-0278-C 01	10	F1.40D	F-A	86E-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0062	ISI-0278-C 01	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0063	ISI-0278-C 01	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0063-IA	ISI-0278-C 01	10	B10.20	B-K-1	86E-02	PT	N-PT-9			1.500	PI,IWA	
RECIR	2-47B408S0064	ISI-0278-C 01	10	F1.40C	F-A	86E-02	VT-3	N-VT-1		28.00		HCF	
RECIR	2-47B408S0065	ISI-0278-C 01	10	F1.40C	F-A	86E-02	VT-3	N-VT-1		28.00		HCF	
RECIR	2-47B408S0066	ISI-0278-C 01	10	F1.40C	F-A	86E-02	VT-3	N-VT-1		28.00		HCF	
RECIR	2-47B408S0067-IE	ISI-0278-C 01	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	CALSTD	COMPDA	NOMTHCK	COMPDESA	COMPDESB
RECIR	2-47B408S0068-IE	ISI-0278-C 01	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0069-IE	ISI-0278-C 01	10	F1.40D	F-A	P10-02	VT-3	N-VT-1				HHS	
RECIR	2-47B408S0070-IA	ISI-0278-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	
RECIR	2-47B408S0071	ISI-0278-C 01	10	F1.10D	F-A	A01-02	VT-3	N-VT-1		22.00		HHS	
RECIR	2-47B408S0071	ISI-0278-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		22.00		HHS	
RECIR	2-47B408S0071-IA	ISI-0278-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			2.000	PI,IWA	
RECIR	2-47B408S0072-IA	ISI-0278-C 01	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	
RECIR	2-47B408S0073	ISI-0278-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		22.00		HHS	
RECIR	2-47B408S0074-IA	ISI-0278-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			1.000	PI,IWA	
RECIR	2-47B408S0075	ISI-0278-C 02	10	F1.10	F-A	A01-02	VT-3	N-VT-1		12.00		HVS	
RECIR	2-47B408S0076	ISI-0278-C 02	10	F1.10D	F-A	86E-02	VT-3	N-VT-1		22.00		HHS	
RECIR	2-47B408S0076	ISI-0278-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		22.00		HHS	
RECIR	2-47B408S0077	ISI-0278-C 02	10	F1.10C	F-A	A01-02	VT-3	N-VT-1		22.00		HVS	
RECIR	2-47B408S0078	ISI-0278-C 02	10	F1.10C	F-A	A01-02	VT-3	N-VT-1		22.00		HHS	
RECIR	2-47B408S0078	ISI-0278-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		22.00		HHS	
RECIR	2-47B408S0081	ISI-0278-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		12.00		HHS	
RECIR	2-47B408S0082	ISI-0278-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		12.00		HHS2	
RECIR	2-47B452S0237	ISI-0278-C 02	10	F1.10D	F-A	86E-02	VT-3	N-VT-1		12.00		HHS	
RECIR	2-47B452S0237	ISI-0278-C 02	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		12.00		HHS	
RECIR	2-47B452S0237-IA	ISI-0278-C 02	10	B10.10	B-K-1	86E-02	PT	N-PT-9			0.625	PI,IWA	
RECIR	GR-2-12	ISI-0270-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-79	12.00	0.569	PBC,SWP	PC,PIPE
RECIR	GR-2-15(OL)	ISI-0270-C 01	10	NU0313	E	B02-02	UT	N-UT-66	BF-50	12.00	1.139	PC,OVLY	
RECIR	GR-2-54	ISI-0270-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	28.00	1.138	PE,EL	PC,PIPE
RECIR	GR-2-60	ISI-0270-C 02	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	28.00	1.138	PE,EL	PC,PIPE
RECIR	GR-2-64(OL)	ISI-0270-C 02	10	NU0313	E	B02-02	UT	N-UT-66	BF-83	28.00	1.750	PC,OVLY	
RECIR	KR-2-02	ISI-0270-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	28.00	1.322	PE,EL	PC,PIPE



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHICK	COMPDESA	COMPDESB
RECIR	KR-2-12	ISI-0270-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	22.00	1.030	PBCC,CR	PC,PIPE
RECIR	KR-2-14	ISI-0270-C 01	10	NU0313	E	D03-02	UT	N-UT-64	BF-88	22.00	1.030	PC,PIPE	PBC,A
RECIR	KR-2-33	ISI-0270-C 02	10	NU0313	C	B02-02	UT	N-UT-64	BF-44	22.00	1.850	PBCC,CR	PC,PIPE
RECIR	KR-2-36	ISI-0270-C 02	10	NU0313	E	D03-02	UT	N-UT-64	BF-88	22.00	1.030	PC,PIPE	PBC,A
RECIR	KR-2-37	ISI-0270-C 02	10	NU0313	E	D03-02	UT	N-UT-64	BF-88	22.00	1.030	PC,PIPE	PC,CAP
RECIR	KR-2-41	ISI-0270-C 02	10	NU0313	E	D03-02	UT	N-UT-64	BF-88	22.00	1.030	PC,PIPE	PBC,A
RECIR	KR-2-45	ISI-0270-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	28.00	1.138	PC,PIPE	PE,EL
RECIR	KR-2-46	ISI-0270-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	28.00	1.138	PC,TEE	PC,PIPE
RECIR	KR-2-47	ISI-0270-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	28.00	1.138	PC,PIPE	PE,EL
RECIR	KR-2-51	ISI-0270-C 02	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	28.00	1.138	PC,PIPE	PE,EL
RECIR	PMP-A-NUT-2-01	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-02	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-03	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-04	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-05	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-06	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-07	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-08	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-09	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-10	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-11	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-12	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-13	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-14	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-15	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-NUT-2-16	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT-1	N-VT-1		03.00		BCHBLTG	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHED	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RECIR	PMP-A-STUD-2-01	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-01	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-02	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-02	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-03	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-03	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-04	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-04	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-05	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-05	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-06	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-06	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-07	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-07	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-08	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-08	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-09	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-09	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-10	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-10	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-11	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-11	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-12	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-12	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-13	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-13	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RECIR	PMP-A-STUD-2-14	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-14	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-15	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-15	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-16	ISI-0407-C 01	10	B6.180	B-G-1	86E-02	UT	N-UT-37	BF-119	03.00		BCHBLTG	
RECIR	PMP-A-STUD-2-16	ISI-0407-C 01	10	B6.180	B-G-1	V01-02	VT-1	N-VT-1		03.00		BCHBLTG	
RECIR	PMP-A-WASH-2-01	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-02	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-03	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-04	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-05	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-06	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-07	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-08	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-09	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-10	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-11	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-12	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-13	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-14	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-15	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	PMP-A-WASH-2-16	ISI-0407-C 01	10	B6.200	B-G-1	86E-02	VT	N-VT-1		03.00		BPUMP	
RECIR	RBC-2-2-BC	ISI-0270-C 02	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BFLG	
RHRS	2-47B452-1206	ISI-0324-C 10	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		18.00		HHS	
RHRS	2-47B452-1320	ISI-0324-C 12	10	F1.20C	F-A	86E-02	VT-3	N-VT-1		24.00		HVS	
RHRS	2-47B452-1320	ISI-0324-C 12	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		24.00		HVS	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RHRS	2-47B452-416	ISI-0324-C 11	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		18.00		HRH	
RHRS	2-47B452-456	ISI-0324-C 09	10	F1.20A	F-A	86E-02	VT-3	N-VT-1		18.00		HRH	
RHRS	2-47B452-708-IA	ISI-0324-C 10	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452-709-IA	ISI-0324-C 10	10	C3.20	C-C	86E-02	MT	N-MT-6			0.750	PI,IWA	
RHRS	2-47B452-715-IA	ISI-0324-C 11	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452-717	ISI-0324-C 09	10	F1.20C	F-A	86E-02	VT-3	N-VT-1		18.00		HVS2	
RHRS	2-47B452-722	ISI-0324-C 09	10	F1.20C	F-A	86E-02	VT-3	N-VT-1		24.00		HVS2	
RHRS	2-47B452-968-IA	ISI-0324-C 09	10	C3.20	C-C	86E-02	MT	N-MT-6			0.750	PI,IWA	
RHRS	2-47B452-970	ISI-0324-C 08	10	F1.20D	F-A	86E-02	VT-3	N-VT-1		24.00		HMS	
RHRS	2-47B452-970-IA	ISI-0324-C 08	10	C3.20	C-C	86E-02	MT	N-MT-6				PI,IWA	
RHRS	2-47B452-983-IE	ISI-0324-C 09	10	F1.40	F-A	86E-02	VT-3	N-VT-1		06.00		HRH	
RHRS	2-47B452-986	ISI-0324-C 10	10	F1.20B	F-A	86E-02	VT-3	N-VT-1		24.00		HRH	
RHRS	2-47B452H0085-IA	ISI-0324-C 01	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452H0091-IA	ISI-0324-C 02	10	C3.20	C-C	86E-02	MT	N-MT-6			0.875	PI,IWA	
RHRS	2-47B452H0119-IA	ISI-0324-C 05	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452H0121-IA	ISI-0324-C 05	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452H0122-IA	ISI-0324-C 05	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452H0124-IA	ISI-0324-C 07	10	C3.20	C-C	86E-02	MT	N-MT-6			0.875	PI,IWA	
RHRS	2-47B452H0126-IA	ISI-0324-C 07	10	C3.20	C-C	86E-02	MT	N-MT-6			0.875	PI,IWA	
RHRS	2-47B452H0128-IA	ISI-0324-C 07	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452R0051	ISI-0324-C 04	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		20.00		HMS2	
RHRS	2-47B452S0113	ISI-0276-C 01	10	F1.10B	F-A	86E-02	VT-3	N-VT-1		20.00		HRH2	
RHRS	2-47B452S0152-IA	ISI-0324-C 05	10	C3.20	C-C	86E-02	MT	N-MT-6			0.375	PI,IWA	
RHRS	2-47B452S0157	ISI-0324-C 08	10	F1.20D	F-A	R16-02	VT-3	N-VT-1		24.00		HHS	
RHRS	2-47B452S0227	ISI-0276-C 01	10	F1.10D	F-A	86E-02	VT-3	N-VT-1		24.00		HHS	
RHRS	2-47B452S0227	ISI-0276-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		24.00		HHS	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHICK	COMPDESA	COMPDESB
RHRS	2-47B452S0229	ISI-0276-C 01	10	F1.10C	F-A	86E-02	VT-3	N-VT-1		24.00		HVS	
RHRS	2-47B452S0234	ISI-0276-C 01	10	F1.10C	F-A	86E-02	VT-3	N-VT-1		24.00		HVS	
RHRS	2-47B452S0235	ISI-0276-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		24.00		HHS	
RHRS	2-47B452S0240	ISI-0276-C 01	10	F1.10D	F-A	P10-02	VT-3	N-VT-1		20.00		HHS	
RHRS	2-47B452S0241-IA	ISI-0324-C 05	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452S0250	ISI-0324-C 09	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		24.00		HHS	
RHRS	2-47B452S0251	ISI-0324-C 09	10	F1.20D	F-A	P10-02	VT-3	N-VT-1		24.00		HHS	
RHRS	2-47B452S0251-IA	ISI-0324-C 09	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-47B452S0274-IA	ISI-0324-C 09	10	C3.20	C-C	86E-02	MT	N-MT-6			1.000	PI,IWA	
RHRS	2-SHV-074-0754		10	B15.70	B-P	R27-02	VT-1	N-VT-1				BVAL	
RHRS	DRHR-2-03	ISI-0221-C 01	10	NU0313	D	B02-02	UT	N-UT-64	BF-57	24.00	1.219	PC,VLV	PC,PIPE
RHRS	DRHR-2-03B	ISI-0221-C 01	10	NU0313	G	B02-02	VT-2	N-VT-4		24.00	1.219	PC,PIPE	PP,PIPE
RHRS	DRHR-2-04	ISI-0221-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	24.00	1.219	PP,PIPE	PE,EL
RHRS	DRHR-2-06	ISI-0221-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	24.00	1.219	PC,VLV	PC,PIPE
RHRS	DRHR-2-11	MSG-0018-C 09	10	NU0313	D	B02-02	UT	N-UT-64	BF-102	24.00	1.531	PC,PIPE	PC,VLV
RHRS	DRHR-2-13B	ISI-0221-C 01	10	NU0313	G	B02-02	VT-2	N-VT-4		24.00	1.219	PC,PIPE	PP,PIPE
RHRS	DRHR-2-16	ISI-0221-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	24.00	1.219	PE,EL	PC,VLV
RHRS	DRHR-2-21	ISI-0221-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-87	20.00	1.031	PE,EL	PC,VLV
RHRS	DSRHR-2-01	ISI-0221-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-57	24.00	1.219	PE,EL	PC,PIPE
RHRS	FCV-74-68-BC	ISI-0221-C 01	10	B7.70	B-G-2	86E-02	VT-1	N-VT-1				BVAL	
RHRS	RHRG-2-06A-A	ISI-0406-C 01	10	C2.31	C-B	86E-02	MT	N-MT-6				VSHELL	VNOZ
RHRS	RHRG-2-06B-A	ISI-0406-C 01	10	C2.31	C-B	86E-02	MT	N-MT-6				VNOZ	
RHRS	RHRG-2-08-A	ISI-0406-C 01	10	C1.10	C-A	86E-02	UT	N-UT-18	BF-40			VSHELL	
RHRS	RHRG-2-10-A	ISI-0406-C 01	10	C1.20	C-A	86E-02	UT	N-UT-18	BF-40			VHEAD	
RHRS	RHRG-2-12-A	ISI-0406-C 01	10	F1.40B	F-A	S01-02	VT-3	N-VT-1				HVES	
RHRS	RHRG-2-12-C	ISI-0406-C 01	10	F1.40B	F-A	S01-02	VT-3	N-VT-1				HVES	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RHRS	RHRG-2-12-C-IA	ISI-0406-C 01	10	C3.10	C-C	86E-02	MT	N-MT-6				PI,IWA	
RHRS	RHRG-2-13-A	ISI-0406-C 01	10	F1.40B	F-A	S01-02	VT-3	N-VT-1				HVES	
RHRS	RHRG-2-14-D	ISI-0406-C 01	10	F1.40B	F-A	S01-02	VT-3	N-VT-1				HVES	
RHRSW	2-47B450H0033-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.188	PI,IWA	
RHRSW	2-47B450H0034-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.322	PI,IWA	
RHRSW	2-47B450H0035-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.322	PI,IWA	
RHRSW	2-47B450H0036-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.188	PI,IWA	
RHRSW	2-47B450H0040	ISI-0145-C 02	10	F1.30C	F-A	86E-02	VT-3	N-VT-1	16.00			HRH	
RHRSW	2-47B450H0040-IA	ISI-0145-C 02	10	D2.40	D-B	86E-02	VT-3	N-VT-1			0.625	PI,IWA	
RHRSW	2-47B450H0041-IA	ISI-0145-C 02	10	D2.40	D-B	86E-02	VT-3	N-VT-1			0.500	PI,IWA	
RHRSW	2-47B450H0042	ISI-0145-C 02	10	F1.30C	F-A	86E-02	VT-3	N-VT-1	12.00			HRH	
RHRSW	2-47B450H0044-IA	ISI-0145-C 03	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.750	PI,IWA	
RHRSW	2-47B450H0049-IA	ISI-0145-C 03	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.438	PI,IWA	
RHRSW	2-47B450R0023-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			1.000	PI,IWA	
RHRSW	2-47B450R0024-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.500	PI,IWA	
RHRSW	2-47B450R0027-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			1.000	PI,IWA	
RHRSW	2-47B450R0029-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.750	PI,IWA	
RHRSW	2-47B450R0030-IA	ISI-0145-C 02	10	D2.20	D-B	86E-02	VT-3	N-VT-1			1.000	PI,IWA	
RHRSW	2-47B450R0035-IA	ISI-0145-C 03	10	D2.20	D-B	86E-02	VT-3	N-VT-1			0.500	PI,IWA	
RHRSW	2-47B450R0040-IA	ISI-0145-C 03	10	D2.20	D-B	86E-02	VT-3	N-VT-1			1.000	PI,IWA	
RPV	2-SI-3.3.01A	N/A	01	10	B15.10	B-P	86E-02	VT-2	N-VT-4			E-LEAK	
RPV	2-SI-3.3.01A	N/A	01	10	B15.50	B-P	86E-02	VT-2	N-VT-4			E-LEAK	
RPV	2-SI-3.3.01A	N/A	01	10	B15.60	B-P	86E-02	VT-2	N-VT-4			E-LEAK	
RPV	2-SI-3.3.01A	N/A	01	10	B15.70	B-P	86E-02	VT-2	N-VT-4			E-LEAK	
RPV	ACCHCOV-1	N/A	00	10	INT	RPV	B06-02	UT	UTBFN211			SHROUD	
RPV	ACCHCOV-2	N/A	00	10	INT	RPV	B06-02	UT	UTBFN211			SHROUD	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDI	NOMTHCK	COMPDESA	COMPDESB
RPV	CORE-SUP-STR	N/A 00	10	B13.40	B-N-2	86E-02	VT-3	VENDOR				SINT	
RPV	CRDN-2-0631-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-1019-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-1051-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	CRDN-2-1427-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-1435-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-1827-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-1831-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-1843-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-2231-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-2247-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-2619-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-2655-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-3043-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-3047-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-3403-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-3411-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-3443-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-3803-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-3819-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-3851-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-4259-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-4635-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-4655-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-5027-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-5043-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RPV	CRDN-2-5415-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-5435-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	PT	N-PT-9				BBOLTS	
RPV	CRDN-2-5435-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	CRDN-2-5843-BC	ISI-0292-C 01	10	B7.80	B-G-2	86E-02	VT-1	N-VT-1				BBOLTS	
RPV	JPRISBR-2-11	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-12	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-13	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-14	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-15	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-16	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-17	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-18	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-19	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	JPRISBR-2-20	N/A 00	10	INT	RPV	B06-02	VT-3	VTBFN204				SRISER	
RPV	N6B-2-2-BC	ISI-0408-C 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BFLG	
RPV	N6B-IR	ISI-0408-C 01	10	B3.100	B-D	86E-02	UT	N-UT-55	BF-81	06.00	4.340	SNIRHS	
RPV	N6B-NV	ISI-0408-C 01	10	B3.90	B-D	86E-02	UT	N-UT-9	BF-19	06.00	0.000	SNOZHS	
RPV	N7-2-1	ISI-0408-C 01	10	B9.11	B-J	86E-02	MT	N-MT-6		04.00	0.780	SNOZHS	PF,FLG
RPV	N7-2-1	ISI-0408-C 01	10	B9.11	B-J	86E-02	UT	N-UT-18	BF-30	04.00	0.780	SNOZHS	PF,FLG
RPV	N7-2-3-BC	ISI-0408-C 01	10	B7.50	B-G-2	86E-02	VT-1	N-VT-1				BFLG	
RPV	N7-IR	ISI-0408-C 01	10	B3.100	B-D	86E-02	UT	N-UT-55	BF-81	04.00	0.000	SNIRHS	
RPV	N7-NV	ISI-0408-C 01	10	B3.90	B-D	86E-02	UT	N-UT-9	BF-19	04.00	4.310	SNOZHS	
RPV	RCH-2-2C	ISI-0408-C 01	10	B1.40	B-A	86E-02	UT	N-UT-9	BF-19			VHEAD	VFLG
RPV	RCH-2-2C-FLEX	ISI-0408-C 01	10	B1.40	B-A	86E-02	MT	N-MT-6				SFLEX	
RPV	RCH-2-2C-FLEX	ISI-0408-C 01	10	B1.40	B-A	V01-02	MT	N-MT-6				SFLEX	
RPV	RCRD-2-33	ISI-0272-C 01	10	NU0313	D	B02-02	UT	N-UT-64	BF-60	04.00	0.750	PC,NOZ	PC,CAP



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RPV	RCRD-2-33	ISI-0272-C 01	10	NU0313	D	B02-02	UT	N-UT-64	BF-76	04.00	0.750	PC,NOZ	PC,CAP
RPV	RPV INT ATT BLR	CHM-2046-C 02	10	B13.20	B-N-2	86E-02	VT-3	VENDOR				SINTATT	
RPV	RPV INT ATT NBLR	CHM-2046-C 02	10	B13.30	B-N-2	86E-02	VT-3	VENDOR				SINTATT	
RPV	RPV-BUSH-2-32	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-33	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-34	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-61	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-62	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-63	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-64	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-65	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-66	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-67	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-68	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-69	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-70	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-71	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-72	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-73	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-74	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-75	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-76	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-77	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-78	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-79	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-80	ISI-0266-C 01	10	B6.50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDA	NOMTHICK	COMPDESA	COMPDESB
RPV	RPV-BUSH-2-81	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-82	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-83	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-84	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-85	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-86	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-87	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-88	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-89	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-90	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-91	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-BUSH-2-92	ISI-0266-C 01	10	B6. 50	B-G-1	86E-02	VT-1	N-VT-1				BCHBLTG	
RPV	RPV-INTERIOR	CHM-2046-C 02	10	B13.10	B-N-1	86E-02	VT-3	VENDOR				SINT	
RPV	RPV-INTERIOR	CHM-2046-C 02	10	712111	RPV	B06-02	EVT	VENDOR				SINT	
RPV	RPV-INTERIOR	CHM-2046-C 02	10	712112	RPV	B06-02	EVT	VENDOR				SINT	
RPV	RPV-INTERIOR	CHM-2046-C 02	10	712115	RPV	B06-02	EVT	VENDOR				SINT	
RPV	RPV-INTERIOR	CHM-2046-C 02	10	712119	RPV	B06-02	EVT	VENDOR				SINT	
RPV	RPV-LIGS-2-01	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-02	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-03	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-04	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-05	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-06	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-07	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-34	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-35	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RPV	RPV-LIGS-2-36	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-37	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-69	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-70	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-71	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-72	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-73	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-74	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-75	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-76	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-77	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-78	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-79	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-80	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-81	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-82	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-83	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-84	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-85	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-86	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-87	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-88	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-89	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-90	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-91	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	
RPV	RPV-LIGS-2-92	ISI-0266-C 01	10	B6. 40	B-G-1	86E-02	UT	N-UT-37	BF-126			BLIG	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHICK	COMPDESA	COMPDESB
RPV	RPV-STUDS-2-09	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-11	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-13	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-15	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-17	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-18	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-19	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-21	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-22	ISI-0266-C 01	10	B6. 30	B-G-1	86E-02	MT	N-MT-6		06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-22	ISI-0266-C 01	10	B6. 30	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-23	ISI-0266-C 01	10	B6. 30	B-G-1	86E-02	MT	N-MT-6		06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-23	ISI-0266-C 01	10	B6. 30	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-24	ISI-0266-C 01	10	B6. 30	B-G-1	86E-02	MT	N-MT-6		06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-24	ISI-0266-C 01	10	B6. 30	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-25	ISI-0266-C 01	10	B6. 30	B-G-1	86E-02	MT	N-MT-6		06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-25	ISI-0266-C 01	10	B6. 30	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-27	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-29	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-31	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-33	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-35	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-37	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-39	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-41	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-42	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-45	ISI-0266-C 01	10	B6. 20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPDIA	NOMTHCK	COMPDESA	COMPDESB
RPV	RPV-STUDS-2-47	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-48	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-49	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-50	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-51	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-53	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-55	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-57	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-59	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-72	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-90	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-91	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-STUDS-2-92	ISI-0266-C 01	10	B6.20	B-G-1	86E-02	UT	N-UT-37	BF-126	06.00	65.56	BCHBLTG	
RPV	RPV-SUPP-2-1	ISI-0415-C 02	10	F1.40B	F-A	86E-02	VT-3	N-VT-1				HVES	
RPV	RPV-SUPP-2-1-IA	ISI-0415-C 02	10	B8.10	B-H	86E-02	MT	N-MT-6				HVES	
RPV	TOPGUIDE	N/A 00	10	INT	RPV	B06-02	VT	VTBFN204				SHROUD	
RWCUS	2-47B406S0018	ISI-0274-C 01	10	F1.10C	F-A	86E-02	VT-3	N-VT-1		06.00		HVS	
RWCUS	2-47B406S0020	ISI-0274-C 01	10	F1.10B	F-A	86E-02	VT-3	N-VT-1		06.00		HRH	
RWCUS	DSRWC-2-03(OL)	ISI-0272-C 01	10	NU0313	E	B02-02	UT	N-UT-66	BF-80	06.00	0.700	PC,OVLY	
RWCUS	DSRWC-2-03(OL)	ISI-0272-C 01	10	TS3432	B-J	B04-02	PT	N-PT-9		06.00	0.700	PC,OVLY	
RWCUS	DSRWC-2-03(OL)	ISI-0272-C 01	10	TS3432	B-J	B04-02	UT	N-UT-66	BF-62	06.00	0.700	PC,OVLY	
RWCUS	DSRWC-2-04(OL)	ISI-0272-C 01	10	NU0313	E	B02-02	UT	N-UT-66	BF-80	06.00	0.700	PC,OVLY	
RWCUS	DSRWC-2-05(OL)	ISI-0272-C 01	10	NU0313	E	B02-02	UT	N-UT-66	BF-62	06.00	0.700	PC,OVLY	
RWCUS	DSRWC-2-05(OL)	ISI-0272-C 01	10	TS3432	B-J	B04-02	PT	N-PT-9		06.00	0.700	PC,OVLY	
RWCUS	DSRWC-2-05(OL)	ISI-0272-C 01	10	TS3432	B-J	B04-02	UT	N-UT-66	BF-80	06.00	0.700	PC,OVLY	
RWCUS	DSRWC-2-06	ISI-0272-C 01	10	NU0313	C	B02-02	UT	N-UT-64	BF-31	06.00	0.432	PE,EL	PC,PIPE



SYSTEM	WELDNO	ISONO	CYCLE	ITEMNO	CATEGORY	EXREQ	EXSCHD	NDEPROC	CALSTD	COMPdia	NOMTHCK	COMPDESA	COMPDESB
RWCUS	DSRWC-2-06	ISI-0272-C 01	10	TS3432	B-J	B04-02	PT	N-PT-9		06.00	0.432	PE,EL	PC,PIPE
RWCUS	DSRWC-2-06	ISI-0272-C 01	10	TS3432	B-J	B04-02	UT	N-UT-64	BF-31	06.00	0.432	PE,EL	PC,PIPE
RWCUS	FCV-69-001	ISI-0272-C 01	10	B12.50	B-M-2	P10-02	VT-3	N-VT-1		06.00		AVALINT	
RWCUS	FCV-69-002	ISI-0272-C 01	10	B12.50	B-M-2	P10-02	VT-3	N-VT-1		06.00		AVALINT	
RWCUS	RWCU-2-003-025	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,VLV	PC,VLV
RWCUS	RWCU-2-003-025	ISI-0272-C 01	10	B9.11	B-J	P10-02	UT	N-UT-18	BF-61			PC,VLV	PC,VLV
RWCUS	RWCU-2-003-026	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,VLV	PC,VLV
RWCUS	RWCU-2-003-026	ISI-0272-C 01	10	B9.11	B-J	P10-02	UT	N-UT-18				PC,VLV	PC,VLV
RWCUS	RWCU-2-003-027	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,EL	PC,VLV
RWCUS	RWCU-2-003-027	ISI-0272-C 01	10	B9.11	B-J	P10-02	UT	N-UT-18	BF-61			PC,EL	PC,VLV
RWCUS	RWCU-2-003-037	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,VLV	PC,VLV
RWCUS	RWCU-2-003-038	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,VLV	PC,VLV
RWCUS	RWCU-2-003-039	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9		04.00	0.337	PC,VLV	PC,VLV
RWCUS	RWCU-2-003-040	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9		04.00	0.337	PC,VLV	PC,VLV
RWCUS	RWCU-2-003-041	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9				PC,VLV	PC,VLV
RWCUS	RWCU-2-003-042	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9		04.00	0.337	PC,EL	PC,EL
RWCUS	RWCU-2-003-043	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9		04.00	0.337	PC,VLV	PC,VLV
RWCUS	RWCU-2-003-044	ISI-0272-C 01	10	B9.11	B-J	P10-02	PT	N-PT-9		06.00		PC,PIPE	PC,PIPE
RWCUS	RWCU-2-003-044	ISI-0272-C 01	10	B9.11	B-J	P10-02	UT	N-UT-18	BF-61	06.00		PC,PIPE	PC,PIPE



ENCLOSURE 2

TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT (BFN)
UNIT 2
AMERICAN SOCIETY OF MECHANICAL ENGINEERS (ASME),
SECTION XI, REPAIR AND REPLACEMENTS PROGRAM
SUMMARY REPORT (NIS-2) FOR CYCLE 10 OPERATION

(SEE ATTACHED)



BROWNS FERRY
NUCLEAR PLANT

UNIT 2 CYCLE 10

ASME SECTION XI

NIS-2 DATA REPORT



OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

APPENDIX I _____ **Summary of Repair and
Replacement Activities**

APPENDIX II _____ **Form NIS-2 Owner's Report
For Repairs or Replacements**



Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

APPENDIX I

SUMMARY OF REPAIR AND REPLACEMENT ACTIVITIES



Owner: TENNESSEE VALLEY AUTHORITY
 1101 Market Street
 Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
 P. O. Box 2000
 Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

<u>WID</u>	<u>SYS</u>	<u>ORG</u>	<u>CLASS</u>	<u>ACTIVITY</u>
DCN T32610	69	TVA	1	REPLACEMENT
DCN T40617			1	
DCN W40713	71	TVA	1	REPLACEMENT
97-004682-000	73	MAINT	2	REPLACED
97-006387-068	1	MAINT	1	REPLACED
97-006387-072			1	
98-003269-000	71	MAINT	2	REPLACED
98-011466-000	3	MAINT	1	REPLACED
98-011553-000			1	
98-011555-000	74	MAINT	1	REPLACED
98-012561-000			1	
98-012563-000			1	
98-013616-000			1	
98-013696-000			1	
98-013698-000			1	
99-003435-001			2	
99-003436-017			2	
99-003436-034			2	
99-003437-001			2	
98-011689-000	1	MAINT	1	REPLACED
98-012621-000			1	
98-012646-000			1	
98-012651-000			1	
98-012655-000			1	
98-013404-000			1	
98-013701-000			1	
98-013706-000			1	
98-012557-000	75	MAINT	1	REPLACED
98-012567-000	73	MAINT	2	REPLACED
98-012568-000	71	MAINT	2	REPLACED



Owner: TENNESSEE VALLEY AUTHORITY
 1101 Market Street
 Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
 P. O. Box 2000
 Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

<u>WID</u>	<u>SYS</u>	<u>ORG</u>	<u>CLASS</u>	<u>ACTIVITY</u>
98-012685-000	68	MAINT	1	REPLACED
98-012807-000				
98-012873-000				
98-012877-000				
98-012910-000				
98-012911-000				
98-012927-000				
98-013024-000				
98-013723-000				
98-013724-000				
98-013726-000				
98-013727-000				
98-013728-000				
98-013730-000				
98-013731-000				
98-013805-000				
98-013824-000				
98-013825-000				
98-013828-000				
98-013558-000	1	MAINT	1	REPLACED
98-013625-000				
98-014051-000	71	MAINT	2	REPLACED
98-015377-000	85	MAINT	1	REPLACEMENT
99-000228-000	75	MAINT	1	REPAIR
99-000237-000	1	MAINT	1	REPLACEMENT
99-000512-000	68	MAINT	1	REPLACED
99-004328-000	85	MAINT	2	REPLACED
99-005552-000	1	TVA	2	REPLACED



Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

LEGEND

WID - Work Implementing Document

Example: A99999 refers to a Design Change Notice
99-999999-999 refers to a Work Order
A99999 refers to a Design Change Notice

SYS- System

1 - Main Steam	68 - Reactor Water Recirculation
3 - Reactor Feedwater	69 - Reactor Water Cleanup
8 - Turbine Drains	71 - Reactor Core Isolation Cooling
10 - Reactor Drains, Vents, and Blowdown	73 - High Pressure Coolant Injection
12 - Auxiliary Boiler	74 - Residual Heat Removal
63 - Standby Liquid Control	75 - Core Spray
	85 - Control Rod Drive
	92 - Neutron Monitoring

ORG - Organization which performed the WID

MAINT - TVA's Maintenance Organization

GE - General Electric Company

TVA - Work performed by Stone and Webster Engineering Corporation
utilizing TVA's Quality Assurance Program and procedures

CLASS - Refers to ASME Code Class 1 or 2

ACTIVITY - Classifies work activity as being repair or replacement as denoted on NIS-2 Form



Owner: TENNESSEE VALLEY AUTHORITY
1101 Market Street
Chattanooga, TN 37402-2801

Plant: Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, AL 35609-2000

Unit: Two

Certificate of Authorization: Not Required

Commercial Service Date: March 1, 1975

National Board Number for Unit: Not Required

APPENDIX II

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS



FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date August 9, 1998

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order 97-004682-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 073, High Pressure Coolant Injection (HPCI) System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
HPCI Turbine Exhaust Inner Rupture Disc	Fike Metal Products 16-CPV-C-BT	Lot # 9723640	N/A	2-RPD-073-0729	1997	Replaced	No

7. Description of Work Replaced rupture disc on turbine exhaust.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi. Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contracts 70C53-92291-7, 221688 and Design Criteria BFN-50-7073 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks This activity was scheduled as a normal, periodic replacement and inspection of the rupture disc as required by ASME/ANSI OM-10.
Applicable Manufacturer's Data Reports to be attached
During inspection of the rupture disc assembly (which had been in service), cracks were discovered on the inner rupture diaphragm. Further
investigation determined the cause to be the old manufacturing method of that series of discs. The disc was manufactured as a two ply element.
The manufacturer (Fike) has confirmed that the rupture discs currently installed are single ply elements. TVA has taken actions to ensure that
only single ply assemblies are purchased in the future.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the
repair or replacement
 ASME Code Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed *[Signature]* System Engineer
Owner or Owner's Designee, Title

Date 8-18 19 98

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State
 or Province of TENN. and employed by HARTFORD SIM. SUP. INSD. & TNS. CO. of
HARTFORD, CONN. have inspected the components described
 in this Owner's Report during the period 5-19-97 to 11-20-98, and state that
 to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's
 Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the
 examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in
 any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
 Inspector's Signature

Commissions

9635 TN. - A-N-T.
 National Board, State, Province, and Endorsements

Date 11-20 19 98

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date August 9, 1998

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order 98-003269-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 071, Reactor Core Isolation Cooling (RCIC) System

5. (a) Applicable Construction Code USAS B31.1:0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
RCIC Turbine Exhaust Rupture Disc	Fike Metal Products 8-CPV-C-BT	13847401	N/A	2-RPD-071-0011A	1989	Replaced	No

7. Description of Work Replaced rupture disc on turbine exhaust.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 70C53-92291-7, 89NJL80177B (ref. PEG pkg 9600000598) and Design Criteria BFN-50-7071 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks This activity is a normal, periodic replacement and inspection of the rupture disc required by ASME/ANSI OM-10 and controlled by
the BFN Preventive Maintenance Program. This rupture disc was replaced early as a conservative action following identified degradation on a
Unit 2 HPCI rupture disc (ref. BFER 980479). No degradation was identified during inspection of the rupture disc removed during this
subject activity.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the
ASME Code Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Stephen C. Willard System Engineer
Owner or Owner's Designee, Title

Date 8-18 19 98

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State
or Province of TENN and employed by HSBTEI of
HARTFORD, CT have inspected the components described
in this Owner's Report during the period 6-19-98 to 6-25-98, and state that
to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's
Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the
examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in
any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd
Inspector's Signature

Commissions

TN 3135 "N" "I"
National Board, State, Province, and Endorsements

Date Aug. 19 19 98

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date March 16, 1999

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order 98-014051-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 071, Reactor Core Isolation Cooling (RCIC) System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
RCIC Turbine Stop Valve	Gimpel Machine Works	N/A	N/A	2-FCV-071-0009	N/A	Replaced	No

7. Description of Work Replaced Unit 2 valve with the Unit 1 valve from the same application. Same manufacturer and valve pattern.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

*IWA 5214(e)
 04/3/24/99
 SCW 3/24/99*

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Design Criteria BFN-50-7071 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Replaced Unit 2 valve with the Unit 1 valve from the same application. Same manufacturer and valve pattern.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE	
We certify that the statements made in the report are correct and this	<u>replacement</u> <small>repair or replacement</small>
conforms to the rules of the ASME Code Section XI.	
Type Code Symbol Stamp	<u>N/A</u>
Certificate of Authorization No.	<u>N/A</u>
Expiration Date	<u>N/A</u>
Signed <u><i>[Signature]</i></u>	Date <u>3-16</u> , 19 <u>99</u>
<small>Owner or Owner's Designee, Title</small>	System Engineer

CERTIFICATE OF INSERVICE INSPECTION	
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of <u>Tenn.</u> and employed by <u>HSB I & T</u> of <u>HARTFORD, CT</u> have inspected the components described in this Owner's Report during the period <u>3-5-99</u> to <u>3-7-99</u> , and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.	
By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.	
<u><i>[Signature]</i></u> <small>Inspector's Signature</small>	Commissions <u>TN 313.5 "N" "I"</u> <small>National Board, State, Province, and Endorsements</small>
Date <u>March 26</u> 19 <u>99</u>	

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street.
Chattanooga, TN 37402-2801
Address

Date May 17, 1999

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Design Change Notice (DCN) T39905A Stages 2 and 6
Work Orders (WO) 97-006387-068 and 97-006387-072
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001, Main Steam System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Line A Outboard Isolation Valve	Atwood & Morrill 20851-H-26	2-86A	N/A	2-FCV-001-0015	N/A	Replacement	No
Main Steam Line C Outboard Isolation Valve	Atwood & Morrill 20851-H-26	2-86C	N/A	2-FCV-001-0038	N/A	Replacement	No

7. Description of Work Installed a modified valve poppet, stem and cover plate (bonnet) on both valves. Replaced one bonnet stud and two nuts on 2-FCV-1-15.


8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

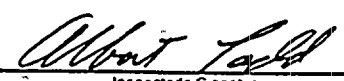
NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contracts 90744, 196744, RD898851 & 224765, GE Purchase Specification 21A1062 Rev. 0 and 21A1062AL Rev. 6 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Installed a modified valve poppet, stem and cover plate (bonnet) into 2-FCV-1-15 & 2-FCV-1-38.
Applicable Manufacturer's Data Reports to be attached
The valve poppet for 2-FCV-1-15 was machined under WVO 98-002608-001. WO 97-006387-068 installed this modification on 2-FCV-1-15 and replaced one bonnet stud and two nuts.
The valve poppet for 2-FCV-1-38 had previously been installed in 3-FCV-1-37. (Note: Both the Unit 2 and Unit 3 MSIVs are Atwood & Morrill 20851-H-26.) That poppet was removed from 3-FCV-1-37 by WO 97-002190-000 and machined by WO 98-014011-000. WO 97-006387-072 installed this modification on 2-FCV-1-38.

CERTIFICATE OF COMPLIANCE			
We certify that the statements made in the report are correct and this ASME Code Section XI.	<u>replacement</u> <small>repair or replacement</small>	conforms to the rules of the	
Type Code Symbol Stamp	<u>N/A</u>		
Certificate of Authorization No.	<u>N/A</u>	Expiration Date	<u>N/A</u>
Signed	 <small>Owner or Owner's Designee, Title</small>	System Engineer	Date <u>5-23</u> , 19 <u>99</u>

CERTIFICATE OF INSERVICE INSPECTION			
I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of <u>TENNESSEE</u> and employed by <u>HEBIEE</u> of <u>HARTFORD, CT</u> have inspected the components described in this Owner's Report during the period <u>4-7-99</u> to <u>5-9-99</u> , and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.			
By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.			
 Inspector's Signature	Commissions	<u>TN 3135 IB SNA</u> <small>National Board, State, Province, and Endorsements</small>	
Date	<u>July 3</u>	19	<u>99</u>

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date May 17, 1999
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Work Orders 98-013558-000 & 98-013625-000
Repair Organization P.O. No., Job No., etc

3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 001, Main Steam System

5. (a) Applicable Construction Code ASME Section III 19 68 Edition, Summer 1970 Addenda, N/A Code Case
ASME Section III, Article 9, 1965 and

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Main Steam Relief Valve	Target Rock Corp. 7567F-000-10	1016	N/A	2-PCV-001-0004	1968	Replaced	No
Main Steam Relief Valve	Target Rock Corp. 7567F-000-10	1030	N/A	2-PCV-001-0004	1968	Replacement	No
Main Steam Relief Valve	Target Rock Corp. 7567F-000-10	1031	N/A	2-PCV-001-0023	1968	Replaced	No
Main Steam Relief Valve	Target Rock Corp. 7567F-000-10	1024	N/A	2-PCV-001-0023	1968	Replacement	No

7. Description of Work Replaced Main Steam Relief valve main bodies. Replaced the inlet and outlet bolting.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in GE P. O. 205AJ600, and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. **Remarks** The main valve bodies were replaced with rebuilt valve bodies previously used on Unit 3 (same manufacturer/model number).

Applicable Manufacturer's Data Reports to be attached

As a part of the Tech Spec required valve inspections WO 98-013558-000 replaced 2-PCV-001-0004 with a rebuilt valve previously used
in BFN Unit 3 (3-PCV-001-0018, S/N 1030) and installed new inlet and outlet flange studs and nuts. The replacement valve was removed
from Unit 3 by WO 98-006535-000 and rebuilt by WO 98-006535-001. Inlet studs and outlet studs and nuts were replaced for
ease of maintenance and ALARA considerations.

Due to suspected seat leakage either through the main seat or the pilot seat WO 98-013625-000 replaced 2-PCV-001-0023 with a rebuilt valve
body previously used in BFN Unit 3 (3-PCV-001-0179, S/N 1024) and installed new inlet and outlet flange studs and nuts. The replacement
valve was removed from Unit 3 by WO 98-006534-000 and rebuilt by WO 98-006534-001. Inlet and outlet studs and nuts were replaced
for ease of maintenance and ALARA considerations.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the
repair or replacement
 ASME Code Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed *Stephen C. Willard*, System Engineer
Owner or Owner's Designee, Title

Date 10-8, 19 99

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State
 or Province of TENNESSEE and employed by HSBT & T of
HARTFORD, CT have inspected the components described
 in this Owner's Report during the period 4/13/99 to 5/8/99, and state that
 to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's
 Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the
 examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in
 any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd
Inspector's Signature

Commissions TN 3135 1B SNA
National Board, State, Province, and Endorsements

Date June 14 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date May 17, 1999
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
4. Identification of System System 001, Main Steam System
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

Work Orders (WO) 98-011689-000, 98-012621-000, 98-012646-000, 98-012651-000, 98-012655-000, 98-013404-000, 98-013701-000, 98-013706-000
Repair Organization P.O. No., Job No., etc.

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B400S0110	Bergen-Paterson	TVA Serial No. M0065	N/A	2-SNUB-001-5060	N/A	Replaced	No
Support (Snubber) 2-47B400S0110	Bergen-Paterson	TVA Serial No. M0272	N/A	2-SNUB-001-5060	N/A	Replacement	No
Support (Snubber) 2-47B400S0109	Bergen-Paterson	TVA Serial No. M0412	N/A	2-SNUB-001-5059	N/A	Replaced	No
Support (Snubber) 2-47B400S0109	Bergen-Paterson	TVA Serial No. M0154	N/A	2-SNUB-001-5059	N/A	Replacement	No

Component listing continued on Sheet 2

7. Description of Work Replaced snubbers with like-for-like; these snubbers are ASME Code Class 1 equivalent components.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** See Remarks
 Other ** Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks WO 98-011869-000 (2-SNUB-001-5060)

Applicable Manufacturer's Data Reports to be attached

The original snubber (TVA Serial No. M0065) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. M0272) was previously located in Unit 2 as Main Steam MS SSC6 UNID 2-SNUB-001-5058 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0272) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-001-5060.

WO 98-012621-000 (2-SNUB-001-5059)

The original snubber (TVA Serial No. M0412) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. M0154) was previously located in Unit 3 as Main Steam (MS) MS SSD1 UNID 3-SNUB-001-5069 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0154) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly. This rebuild also required the replacement of the reservoir cylinder tube, with a new one, which was purchased on Contract No. 99NDF246300, Item No. 47. The new snubber was functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-001-5059.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Williams*, System Engineer Date 6-21, 19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSB&I of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/7/99 to 5/9/99 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd
Inspector's Signature

Commissions TN 3135 IB SNA
National Board, State, Province, and Endorsements

Date June 23 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Date May 17, 1999
Name
1101 Market Street
Address
Chattanooga TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 2 of 2
Name
P. O. Box 2000, Decatur, AL 35609-2000 Unit 2
Address
 Work Orders (WO) 98-011689-000, 98-012621-000,
98-012646-000, 98-012651-000, 98-012655-000,
98-013404-000, 98-013701-000, 98-013706-000
3. Work Performed by TVA-BFN Repair Organization P.O. No., Job No., etc
Name Type Code Symbol Stamp N/A
P. O. Box 2000, Decatur, AL 35609-2000 Authorization No. N/A
Address Expiration Date N/A
4. Identification of System System 001 Main Steam System
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B400S0107	Bergen-Paterson	TVA Serial No. M0475	N/A	2-SNUB-001-5057	N/A	Replaced	No
Support (Snubber) 2-47B400S0107	Bergen-Paterson	TVA Serial No. M0290	N/A	2-SNUB-001-5057	N/A	Replacement	No
Support (Snubber) 2-47B400S0108	Bergen-Paterson	TVA Serial No. M0404	N/A	2-SNUB-001-5058	N/A	Replaced	No
Support (Snubber) 2-47B400S0108	Bergen-Paterson	TVA Serial No. M0329	N/A	2-SNUB-001-5058	N/A	Replacement	No
Support (Snubber) 2-47B400S0103	Bergen-Paterson	TVA Serial No. M0061	N/A	2-SNUB-001-5053	N/A	Replaced	No
Support (Snubber) 2-47B400S0103	Bergen-Paterson	TVA Serial No. M0145	N/A	2-SNUB-001-5053	N/A	Replacement	No
Support (Snubber) RSSJ-1	Pacific Scientific	425	N/A	2-SNUB-001-5030	N/A	Replaced	No
Support (Snubber) RSSJ-1	Pacific Scientific	513	N/A	2-SNUB-001-5030	N/A	Replacement	No
Support (Snubber) 2-47B400S0104	Bergen-Paterson	TVA Serial No. M0147	N/A	2-SNUB-001-5054	N/A	Replaced	No
Support (Snubber) 2-47B400S0104	Anchor/Darling	ADH-3000-781	N/A	2-SNUB-001-5054	N/A	Replacement	No
Support (Snubber) 2-47B400S0105	Bergen-Paterson	TVA Serial No. M0074	N/A	2-SNUB-001-5055	N/A	Replaced	No
Support (Snubber) 2-47B400S0105	Anchor/Darling	ADH-3000-782	N/A	2-SNUB-001-5055	N/A	Replacement	No

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

9. Remarks WO 98-012646-000 (2-SNUB-001-5057)

The original snubber (TVA Serial No. M0475) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. M0290) was previously located in Unit 2 as Reactor Building Closed Cooling Water (RBCCW) R3U UNID 2-SNUB-070-5005 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0290) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-001-5057.

WO 98-012651-000 (2-SNUB-001-5058)

The original snubber (TVA Serial No. M0404) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. M0329) was previously located in Unit 2 as Residual Heat Removal (RHR) 2-47B452R0020U (R20U) UNID2-SNUB-074-5007 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0329) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly. This rebuild also required the replacement of the main cylinder tube from a spare snubber (TVA Serial No. M0386) which was previously located in Unit 1 as Residual Heat Removal (RHR) R9N UNID 1-SNUB-074-5040 which had been replaced by MMI-45 and kept as a spare. The snubber was functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-001-5058.

WO 98-012655-000 (2-SNUB-001-5053)

The original snubber (TVA Serial No. M0061) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. M0145) was previously located in Unit 2 as Residual Heat Removal (RHR) (2-47B452R0020 U) UNID 1-SNUB-074-5007 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0145) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-001-5053.

WO 98-013404-000 (2-SNUB-001-5030)

The original snubber (Serial No. 425) was removed and scrapped due to high drag force. The newly installed snubber (Serial No. 513) was located at Unit 1 MSRVP TP 71B2, UNID 2-SNUB-001-5006. The new snubber (Serial No. 513) was functionally tested per 2-SI-4.6.H-2A and found acceptable prior to installation at 2-SNUB-001-5030.

WO 98-013701-000 (2-SNUB-001-5054)

The original snubber (TVA Serial No. M0147) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-3000-781) was purchased under contract 99N2R-244798-000. The new snubber (Serial No. ADH-3000-782) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-001-5054.

WO 98-013706-000 (2-SNUB-001-5055)

The original snubber (TVA Serial No. M0074) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-3000-782) was purchased under contract 99N2R-244798-000. The new snubber (Serial No. ADH-3000-781) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-001-5055.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 3, 1999
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Address
P. O. Box 2000, Decatur, AL 35609-2000
Repair Organization P.O. No., Job No., etc
Work Order 98-011466-000 and 98-011553-000
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 003, Reactor Feedwater (FW) System
5. (a) Applicable Construction Code USAS.B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B415S0012	Bergen-Paterson	TVA Serial No. M0279	N/A	2-SNUB-003-5027	N/A	Replaced	No
Support (Snubber) 2-47B415S0012	Bergen-Paterson	TVA Serial No. M0361	N/A	2-SNUB-003-5027	N/A	Replacement	No
Support (Snubber) 2-47B415S0004	Bergen-Paterson	TVA Serial No. M0460	N/A	2-SNUB-003-5018	N/A	Replaced	No
Support (Snubber) 2-47B415S0004	Bergen-Paterson	TVA Serial No. M0426	N/A	2-SNUB-003-5018	N/A	Replacement	No

7. Description of Work Replaced snubbers 2-SNUB-003-5027 and 2-SNUB-003-5018 with like-for-like; these snubbers are ASME Code Class 1 equivalent components.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** See Remarks
 Other: Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X.11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7003 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks WO 98-011466-000 - (2-SNUB-003-5027) -

Applicable Manufacturer's Data Reports to be attached

The original snubber (TVA Serial No. M0279) was removed and stored in the snubber room as a spare due to seal life expiration.

The newly installed snubber (TVA Serial No. M0361) was previously located in Unit 3 as Residual Heat Removal (RHR), R65

UNID 3-SNUB-074-5060 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0361)

was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and

functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-003-5027.

WO 98-011553-000 - (2-SNUB-003-5018) -

The original snubber (TVA Serial No. M0460) was removed and stored in the snubber room as a spare due to seal life expiration.

The newly installed snubber (TVA Serial No. M0426) was previously located in Unit 1 as Feedwater SSB4, UNID 1-SNUB-003-5027, which had

been replaced per MMI-45 and kept as a spare. The new snubber (TVA Serial No. M0426) was rebuilt per MPI-0-000SNB002, Hydraulic Shock

and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of

Bergen-Paterson Snubbers, prior to installation at 2-SNUB-003-5018.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Stephen C. Williams, System Engineer
Owner or Owner's Designee, Title

Date 6-8, 19 99

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBTEL of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/4/99 to 4/15/99 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert T. Todd
Inspector's Signature

Commissions TN3135 I B SNA
National Board, State, Province, and Endorsements

Date June 17 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 14, 1999

Sheet 1 of 5

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Orders (WO) 98-012685-000, 98-012807-000, 98-012873-000, 98-012877-000, 98-012910-000, 98-012911-000, 98-012927-000, 98-013024-000, 98-013723-000, 98-013724-000, 98-013726-000, 98-013727-000, 98-013728-000, 98-013730-000, 98-013731-000, 98-013805-000, 98-013824-000, 98-013825-000 & 98-013828-000
Repair Organization P.O. No., Job No., etc

3. Work TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 068, Reactor Recirculation System

5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No:	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B408S0058	Grinnell	TVA Serial No. G036	N/A	2-SNUB-068-5018	N/A	Replaced	No
Support (Snubber) 2-47B408S0058	Grinnell	TVA Serial No. G018	N/A	2-SNUB-068-5018	N/A	Replacement	No
Support (Snubber) 2-47B408S0048	Grinnell	TVA Serial No. G001	N/A	2-SNUB-068-5016	N/A	Replaced	No
Support (Snubber) 2-47B408S0048	Grinnell	TVA Serial No. G016	N/A	2-SNUB-068-5016	N/A	Replacement	No

See Sheet 2 for continuation of component listing

7. Description of Work Replaced the referenced snubbers with either new or rebuilt, like-for-like snubbers.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** See Remarks
Other ** Pressure N/A psi Test Temp. N/A *F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

* as amended by additional quality assurances requirements found in contract 68C37-91602, BFN-50-7068 and BFN-50-C-7105

FORM NIS-2 (Back)

9. Remarks WO 98-012685-000 (2-SNUB-068-5018)

Applicable Manufacturer's Data Reports to be attached

The original snubber (TVA Serial No. G036) was removed and stored in the snubber room as a spare.

The newly installed snubber (TVA Serial No. G018) was previously located in Unit 1 as Reactor Recirculation SS8

UNID 1-SNUB-068-5018 and was removed by WO 98-011383-000 per MPI-0-000-SNB004. The newly installed snubber

(TVA Serial No. G018) was rebuilt per MPI-0-000-SNB001, Hydraulic Shock and Sway Arrestor Grinnell Unit Disassembly and

Reassembly, and functionally tested per 2-SI-4.6.H-2D, Functional Testing of Grinnell Hydraulic Snubbers, prior to installation

at 2-SNUB-068-5018. The rear end attachment was used from the Unit 1 snubber.

WOs 98-012807-000 & (2-SNUB-068-5016)

The original snubber (TVA Serial No. G001) was removed and stored in the snubber room as a spare.

The newly installed snubber (TVA Serial No. G016) was previously located in Unit 1 as Reactor Recirculation SS6B

UNID 1-SNUB-068-5016 and was removed by WO 98-011432-000 per MPI-0-000-SNB004. The newly installed snubber

(TVA Serial No. G016) was rebuilt per MPI-0-000-SNB001, Hydraulic Shock and Sway Arrestor Grinnell Unit Disassembly and

Reassembly, and functionally tested per 2-SI-4.6.H-2D, Functional Testing of Grinnell Hydraulic Snubbers, prior to installation

at 2-SNUB-068-5016. The rear end attachment was used from the Unit 1 snubber.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed *Stephen G. Williams* System Engineer
Owner or Owner's Designee, Title

Date 6-21 19 99

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBI & I of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4/7/99 to 5/9/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd
Inspector's Signature

Commissions

TN 3135

I.B.S.N.A

National Board, State, Province, and Endorsements

Date June 29 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Date June 14, 1999
1101 Market Street
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 2 of 5
P. O. Box 2000, Decatur, AL 35609-2000 Unit 2
Name
Address
3. Work Performed by TVA-BFN Work Orders (WO) 98-012685-000, 98-012807-000,
P. O. Box 2000, Decatur, AL 35609-2000 98-012873-000, 98-012877-000, 98-012910-000,
98-012911-000, 98-012927-000, 98-013024-000,
98-013723-000, 98-013724-000, 98-013726-000,
98-013727-000, 98-013728-000, 98-013730-000,
98-013731-000, 98-013805-000, 98-013824-000,
98-013825-000 & 98-013828-000
Repair Organization P.O. No., Job No., etc.
Type Code Symbol Stamp N/A
Authorization No. N/A
Expiration Date N/A
4. Identification of System System 068, Reactor Recirculation System
5. (a) Applicable Construction Code USA'S B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B408S0069	Grinnell	TVA Serial No. G023	N/A	2-SNUB-068-5005	N/A	Replaced	No
Support (Snubber) 2-47B408S0069	Grinnell	TVA Serial No. G011	N/A	2-SNUB-068-5005	N/A	Replacement	No
Support (Snubber) 2-47B408S0067	Grinnell	TVA Serial No. 34340	N/A	2-SNUB-068-5006	N/A	Replaced	No
Support (Snubber) 2-47B408S0067	Grinnell	TVA Serial No. G015	N/A	2-SNUB-068-5006	N/A	Replacement	No
Support (Snubber) 2-47B408S0054	Grinnell	TVA Serial No. 1539D4	N/A	2-SNUB-068-5008	N/A	Replaced	No
Support (Snubber) 2-47B408S0054	Grinnell	TVA Serial No. G002	N/A	2-SNUB-068-5008	N/A	Replacement	No
Support (Snubber) 2-47B408S0068	Grinnell	TVA Serial No. G02	N/A	2-SNUB-068-5009	N/A	Replaced	No
Support (Snubber) 2-47B408S0068	Grinnell	TVA Serial No. G003	N/A	2-SNUB-068-5009	N/A	Replacement	No
Support (Snubber) 2-47B408S0053	Grinnell	TVA Serial No. G028	N/A	2-SNUB-068-5010	N/A	Replaced	No
Support (Snubber) 2-47B408S0053	Grinnell	TVA Serial No. G004	N/A	2-SNUB-068-5010	N/A	Replacement	No

See Sheet 3 for continuation of component listing

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

9. Remarks WO 98-012873-000 (2-SNUB-068-5005)

The original snubber (TVA Serial No. G023) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. G0011) was previously located in Unit 1 as Reactor Recirculation UNID 1-SNUB-068-5004 which was removed by Work Order 98-011462-000. The new snubber (TVA Serial No. G0011) was rebuilt per MPI-0-000-SNB001, Hydraulic Shock and Sway Arrestor Grinnell Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2D, Functional Testing of Grinnell Snubbers, prior to installation at 2-SNUB-068-5005. The rear end attachment was used from the Unit 1 snubber.

WO 98-012877-000 (2-SNUB-068-5006)

The original snubber (TVA Serial No. 34340) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. G015) was previously located in Unit 1 as Reactor Recirculation UNID 1-SNUB-068-5015 and was removed from Unit 1 by Work Order 98-011457-000. The new snubber (TVA Serial No. G015) was rebuilt per MPI-0-000-SNB001, Hydraulic Shock and Sway Arrestor Grinnell Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2D, Functional Testing of Grinnell Hydraulic Snubbers, prior to installation at 2-SNUB-068-5006. The rear end attachment was used from the Unit 1 snubber.

WO 98-012910-000 (2-SNUB-068-5008)

The original snubber (TVA Serial No. 1539D4) was removed and stored in the snubber room as a spare. The newly installed snubber (TVA Serial No. G002) was previously located in Unit 1 as Reactor Recirculation SS1B UNID 1-SNUB-068-5002 and was removed by WO 98-011429-000 per MPI-0-000-SNB004. The newly installed snubber (TVA Serial No. G002) was rebuilt per MPI-0-000-SNB001, Hydraulic Shock and Sway Arrestor Grinnell Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2D, Functional Testing of Grinnell Hydraulic Snubbers, prior to installation at 2-SNUB-068-5008. The rear end attachment was used from the Unit 1 snubber.

WO 98-012911-000 (2-SNUB-068-5009)

The original snubber (TVA Serial No. G02) was removed and scrapped. The newly installed snubber (TVA Serial No. G003) was previously located in Unit 1 as Reactor Recirculation SS2A UNID 1-SNUB-068-5003 and was removed by WO 98-011431-000 per MPI-0-000-SNB004. The newly installed snubber (TVA Serial No. G003) was rebuilt per MPI-0-000-SNB001, Hydraulic Shock and Sway Arrestor Grinnell Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2D, Functional Testing of Grinnell Hydraulic Snubbers, prior to installation at 2-SNUB-068-5009. The rear end attachment was used from the Unit 1 snubber.

WO 98-012927-000 (2-SNUB-068-5010)

The original snubber (TVA Serial No. G028) was removed and stored in the snubber room as a spare. The newly installed snubber (TVA Serial No. G004) was previously located in Unit 1 as Reactor Recirculation SS2B UNID 1-SNUB-068-5004 and was removed by WO 98-011464-000 per MPI-0-000-SNB004. The newly installed snubber (TVA Serial No. G004) was rebuilt per MPI-0-000-SNB001, Hydraulic Shock and Sway Arrestor Grinnell Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2D, Functional Testing of Grinnell Hydraulic Snubbers, prior to installation at 2-SNUB-068-5010.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA)
1101 Market Street
Chattanooga TN 37402-2801

Date June 14, 1999

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Sheet 3 of 5
Unit 2

Work Orders (WO) 98-012685-000, 98-012807-000, 98-012873-000, 98-012877-000, 98-012910-000, 98-012911-000, 98-012927-000, 98-013024-000, 98-013723-000, 98-013724-000, 98-013726-000, 98-013727-000, 98-013728-000, 98-013730-000, 98-013731-000, 98-013805-000, 98-013824-000, 98-013825-000 & 98-013828-000

3. Work Performed by TVA-BFN
P. O. Box 2000, Decatur, AL 35609-2000

Repair Organization P.O. No., Job No., etc.
Type Code Symbol Stamp N/A

Authorization No. N/A
Expiration Date N/A

4. Identification of System System 068 Reactor Recirculation System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B408S0073	Anchor/Darling	ADH-7003-114	N/A	2-SNUB-068-5012	N/A	Replaced	No
Support (Snubber) 2-47B408S0073	Fronek (Anchor/Darling)	ADH-7000-161	N/A	2-SNUB-068-5012	N/A	Replacement	No
Support (Snubber) 2-47B408S0061	Anchor/Darling	ADH-7003-113	N/A	2-SNUB-068-5001	N/A	Replaced	No
Support (Snubber) 2-47B408S0061	Fronek (Anchor/Darling)	ADH-7000-160	N/A	2-SNUB-068-5001	N/A	Replacement	No
Support (Snubber) 2-47B408S0046	Anchor/Darling	ADH-5003-281	N/A	2-SNUB-068-5002	N/A	Replaced	No
Support (Snubber) 2-47B408S0046	Fronek (Anchor/Darling)	ADH-5000-416	N/A	2-SNUB-068-5002	N/A	Replacement	No
Support (Snubber) 2-47B408S0063	Anchor/Darling	ADH-7003-111	N/A	2-SNUB-068-5003	N/A	Replaced	No
Support (Snubber) 2-47B408S0063	Fronek (Anchor/Darling)	ADH-7000-159	N/A	2-SNUB-068-5003	N/A	Replacement	No
Support (Snubber) 2-47B408S0047	Anchor/Darling	ADH-5003-279	N/A	2-SNUB-068-5004	N/A	Replaced	No
Support (Snubber) 2-47B408S0047	Fronek (Anchor/Darling)	ADH-5000-417	N/A	2-SNUB-068-5004	N/A	Replacement	No

See Sheet 4 for continuation of component listing

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

9. Remarks WO 98-013024-000 (2-SNUB-068-5012)

The original snubber (Serial No. ADH-7003-114) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-7000-161) was purchased under contract 99N2R-244798-000.

Applicable Manufacturer's Data Reports to be attached

The new snubber (Serial No. ADH-7000-161) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5012. Fronek purchased Anchor/Darling.

WO 98-013723-000 (2-SNUB-068-5001)

The original snubber (Serial Number ADH-7003-113) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Manufacturer Serial Number ADH-7000-160) was purchased under contract

99N2R-244698-000. The new snubber (Manufacturer Serial Number ADH-7000-160) was functionally tested and found acceptable by the manufacturer, prior to installation at 2-SNUB-068-5001. Fronek purchased Anchor/Darling.

WO 98-013724-000 (2-SNUB-068-5002)

The original snubber (Serial No. ADH-5003-281) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-5000-416) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-5000-416) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5002. Fronek purchased Anchor/Darling.

WO 98-013726-000 (2-SNUB-068-5003)

The original snubber (Serial No. ADH-7003-111) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-7000-159) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-7000-159) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5003. Fronek purchased Anchor/Darling.

WO 98-013727-000 (2-SNUB-068-5004)

The original snubber (Serial No. ADH-5003-279) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-5000-417) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-5000-417) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5004. Fronek purchased Anchor/Darling.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Date June 14, 1999
1101 Market Street
Chattanooga, TN - 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 4 of 5
Name
P. O. Box 2000, Decatur, AL 35609-2000 Unit 2
Address
3. Work Performed by TVA-BFN Work Orders (WO) 98-012685-000, 98-012807-000,
P. O. Box 2000, Decatur, AL 35609-2000 98-012873-000, 98-012877-000, 98-012910-000,
98-012911-000, 98-012927-000, 98-013024-000,
98-013723-000, 98-013724-000, 98-013726-000,
98-013727-000, 98-013728-000, 98-013730-000,
98-013731-000, 98-013805-000, 98-013824-000,
98-013825-000 & 98-013828-000
4. Identification of System System 06B, Reactor Recirculation System
Repair Organization P.O. No., Job No., etc.
Type Code Symbol Stamp N/A
Authorization No. N/A
Expiration Date N/A
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B408S0071	Anchor/Darling	ADH-7003-110	N/A	2-SNUB-068-5011	N/A	Replaced	No
Support (Snubber) 2-47B408S0071	Fronek (Anchor/Darling)	ADH-7000-158	N/A	2-SNUB-068-5011	N/A	Replacement	No
Support (Snubber) 2-47B408S0076	Anchor/Darling	ADH-13003-25	N/A	2-SNUB-068-5013	N/A	Replaced	No
Support (Snubber) 2-47B408S0076	Fronek (Anchor/Darling)	ADH-13000-28	N/A	2-SNUB-068-5013	N/A	Replacement	No
Support (Snubber) 2-47B408S0078	Anchor/Darling	ADH-5003-276	N/A	2-SNUB-068-5014	N/A	Replaced	No
Support (Snubber) 2-47B408S0078	Fronek (Anchor/Darling)	ADH-5000-415	N/A	2-SNUB-068-5014	N/A	Replacement	No
Support (Snubber) 2-47B408S0062	Anchor/Darling	ADH-5003-278	N/A	2-SNUB-068-5015	N/A	Replaced	No
Support (Snubber) 2-47B408S0062	Fronek (Anchor/Darling)	ADH-5000-418	N/A	2-SNUB-068-5015	N/A	Replacement	No
Support (Snubber) 2-47B408S0081	Anchor/Darling	ADH-3003-417	N/A	2-SNUB-068-5021	N/A	Replaced	No
Support (Snubber) 2-47B408S0081	Fronek (Anchor/Darling)	ADH-3002-783	N/A	2-SNUB-068-5021	N/A	Replacement	No

See Sheet 5 for continuation of component listing.

FORM NIS-2; SUPPLEMENTAL SHEET (Back)

9. Remarks WO 98-013728-000 (2-SNUB-068-5011)

The original snubber (Serial No. ADH-7003-110) was removed and stored in the snubber room as a spare due to seal life expiration. ^{Applicable Manufacturer's Data Reports to be attached} The newly installed snubber (Serial No. ADH-7000-158) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-7000-158) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5011. Fronex purchased Anchor/Darling.

WO 98-013730-000 (2-SNUB-068-5013)

The original snubber (Serial No. ADH-13003-25) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-13000-28) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-13000-28) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5013. Fronex purchased Anchor/Darling.

WO 98-013731-000 (2-SNUB-068-5014)

The original snubber (Serial No. ADH-5003-276) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-5000-415) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-5000-415) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5014. Fronex purchased Anchor/Darling.

WO 98-013805-000 (2-SNUB-068-5015)

The original snubber (Serial No. ADH-5003-278) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-5000-418) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-5000-418) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5015. Fronex purchased Anchor/Darling.

WO 98-013824-000 (2-SNUB-068-5021)

The original snubber (Serial No. ADH-3003-417) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-3002-783) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-3002-783) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5021. Fronex purchased Anchor/Darling.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA)
1101 Market Street
Chattanooga, TN 37402-2801

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Date June 14, 1999

Sheet 5 of 5

Unit 2

Work Orders (WO) 98-012685-000, 98-012807-000, 98-012873-000, 98-012877-000, 98-012910-000, 98-012911-000, 98-012927-000, 98-013024-000, 98-013723-000, 98-013724-000, 98-013726-000, 98-013727-000, 98-013728-000, 98-013730-000, 98-013731-000, 98-013805-000, 98-013824-000, 98-013825-000 & 98-013828-000

3. Work Performed by TVA-BFN
P. O. Box 2000, Decatur, AL 35609-2000

Repair Organization P.O. No., Job No., etc.
 Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 068, Reactor Recirculation System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B408S0082	Anchor/Darling	ADH-3003-419	N/A	2-SNUB-068-5022	N/A	Replaced	No
Support (Snubber) 2-47B408S0082	Fronek (Anchor/Darling)	ADH-3002-785	N/A	2-SNUB-068-5022	N/A	Replacement	No
Support (Snubber) 2-47B408S0082	Anchor/Darling	ADH-3003-420	N/A	2-SNUB-068-5023	N/A	Replaced	No
Support (Snubber) 2-47B408S0082	Fronek (Anchor/Darling)	ADH-3002-787	N/A	2-SNUB-068-5023	N/A	Replacement	No

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

9. Remarks WO 98-013825-000 (2-SNUB-068-5022)

The original snubber (Serial No. ADH-3003-419) was removed and stored in the snubber room as a spare due to seal life expiration. Applicable Manufacturer's Data Reports to be attached
The newly installed snubber (Serial No. ADH-3002-785) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-3002-785) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5022. Fronek purchased Anchor/Darling.

WO 98-013828-000 (2-SNUB-068-5023)

The original snubber (Serial No. ADH-3003-420) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (Serial No. ADH-3002-787) was purchased under contract 99N2R-244798-000.

The new snubber (Serial No. ADH-3002-787) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-068-5023. Fronek purchased Anchor/Darling.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner <u>Tennessee Valley Authority (TVA)</u> <small>Name</small> <u>1101 Market Street</u> <u>Chattanooga, TN 37402-2801</u> <small>Address</small>	Date <u>June 2, 1999</u> Sheet <u>1</u> of <u>1</u>
2. Plant <u>Browns Ferry Nuclear Plant (BFN)</u> <u>P. O. Box 2000, Decatur, AL 35609-2000</u> <small>Address</small>	Unit <u>2</u> Work Order <u>98-012568-000</u> <small>Repair Organization P.O. No., Job No., etc</small>
3. Work Performed by <u>TVA-BFN</u> <small>Name</small> <u>P. O. Box 2000, Decatur, AL 35609-2000</u> <small>Address</small>	Type Code Symbol Stamp <u>N/A</u> Authorization No. <u>N/A</u> Expiration Date <u>N/A</u>

4. Identification of System System 071, Reactor Core Isolation Cooling (RCIC) System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) RCIC R4S	Bergen-Paterson	TVA Serial No. M0500	N/A	2-SNUB-071-5005	N/A	Replaced	No
Support (Snubber) RCIC R4S	Bergen-Paterson	TVA Serial No. M0070	N/A	2-SNUB-071-5005	N/A	Replacement	No

7. Description of Work Replaced snubber 2-SNUB-071-5005 with like-for-like;
this snubber is an ASME Code Class 2 equivalent component.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** See Remarks

Other ** Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7071 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks The original snubber (TVA Serial No. M0500) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. M0070) was previously located in Unit 1 as Main Steam (MS) SSC1 UNID 1-SNUB-001-5060 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0070) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-071-5005.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed *Stephen C. Wilford*, System Engineer
Owner or Owner's Designee, Title

Date 6-8, 19 99

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBT&T of HARTFORD, CT have inspected the components described in this Owner's Report during the period 3/24/99 to 3/29/99 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd
Inspector's Signature

Commissions

TN 3135 I B S N A
National Board, State, Province, and Endorsements

Date June 11, 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 2, 1999

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order 98-012567-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 073, High Pressure Coolant Injection (HPCI) System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 * Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B455R0020	Bergen-Paterson	TVA Serial No. M0042	N/A	2-SNUB-073-5004	N/A	Replaced	No
Support (Snubber) 2-47B455R0020	Bergen-Paterson	TVA Serial No. M0430	N/A	2-SNUB-073-5004	N/A	Replacement	No

7. Description of Work Replaced snubber 2-SNUB-073-5004 with like-for-like; this snubber is an ASME Code Class 2 equivalent component.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure **See Remarks
 Other ** Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet; and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7073 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks The original snubber (TVA Serial No. M0042) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. M0430) was previously located in Unit 2 as Residual Heat Removal (RHR) R93W UNID 2-SNUB-074-5021 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0430) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-073-5004.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Stephen C. Willard, System Engineer Date 6-8, 19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBIET of HARTFORD, CT. have inspected the components described in this Owner's Report during the period 2/24/99 to 3/17/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd
Inspector's Signature

Commissions TN 3135 I B N S A
National Board, State, Province, and Endorsements

Date June 11 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 14, 1999
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address

Sheet 1 of 3

2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Work Orders (WO) 98-011555-000, 98-012561-000, 98-012563-000, 98-013616-000, 98-013696-000, 98-013698-000, 99-003435-001, 99-003436-017, 99-003436-034, 99-003437-001
Repair Organization P.O. No., Job No., etc

3. Work TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Authorization No. N/A
 Expiration Date N/A

4. Identification of System System 074, Residual Heat Removal System

5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B452S0250	Bergen-Paterson	TVA Serial No. M0458	N/A	2-SNUB-074-5004	N/A	Replaced	No
Support (Snubber) 2-47B452S0250	Bergen-Paterson	TVA Serial No. M0328	N/A	2-SNUB-074-5004	N/A	Replacement	No
Support (Snubber) 2-47B452S0237	Anchor/Darling	ADH-2003-796	N/A	2-SNUB-074-5057	N/A	Replaced	No
Support (Snubber) 2-47B452S0237	Bergen-Paterson	TVA Serial No. M0221	N/A	2-SNUB-074-5057	N/A	Replacement	No

See Sheet 2 for continuation of component listing

7. Description of Work Replaced the referenced snubbers with either new or rebuilt, like-for-like snubbers.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure ** See Remarks
 Other ** Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

* as amended by additional quality assurances requirements found in contract 68C37-91602, BFN-50-7074 and BFN-50-C-7105

FORM NIS-2 (Back)

9. Remarks WO 98-011555-000 (2-SNUB-074-5004)

The original snubber (TVA Serial No. M0458) was removed and stored in the snubber room as a spare. Applicable Manufacturer's Data Reports to be attached

The newly installed snubber (TVA Serial No. M0328) was previously located in Unit 2 as Residual Heat Removal 2-47B452S0250

UNID 2-SNUB-074-5004 and was removed during the Unit 2 Cycle 5 outage per MMI-45. The newly installed snubber

(TVA Serial No. M0328) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and

Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Hydraulic Snubbers, prior to installation

at 2-SNUB-074-5004.

WO 98-012561-000 (2-SNUB-074-5057)
 The original snubber (Serial No. ADH-2003-796) was removed and stored in the snubber room as a spare due to seal life

expiration. The newly installed snubber (TVA Serial No. M0221) was previously located in Unit 2 as Main Steam (MS) SSA2 Applicable Manufacturer's Data Reports to be attached

UNID 2-SNUB-001-5048 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0221)

was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and

functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-074-5057.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI. repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Williams* System Engineer Date 6-21 1999
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HARTFORD, CT of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4-7-99 to 5-9-99 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd Commissions TN 3135 IBSNA
Inspector's Signature National Board, State, Province, and Endorsements

Date June 24 1999

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
SUPPLEMENTAL SHEET**

1. Owner Tennessee Valley Authority (TVA) Date June 14, 1999
1101 Market Street
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 2 of 3
P. O. Box 2000, Decatur, AL 35609-2000 Unit 2
Name
Address
3. Work Performed by TVA-BFN Work Orders (WO) 98-011555-000, 98-012561-000,
P. O. Box 2000, Decatur, AL 35609-2000 98-012563-000, 98-013616-000, 98-013696-000,
98-013698-000, 99-003435-001, 99-003436-017,
99-003436-034, 99-003437-001
Repair Organization P.O. No., Job No., etc.
Type Code Symbol Stamp N/A
Authorization No. N/A
Expiration Date N/A
4. Identification of System System 074 Residual Heat Removal System
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B452S0235	Anchor/Darling	ADH-2003-795	N/A	2-SNUB-074-5037	N/A	Replaced	No
Support (Snubber) 2-47B452S0235	Bergen-Paterson	TVA Serial No. M0503	N/A	2-SNUB-074-5037	N/A	Replacement	No
Support (Snubber) 2-47B452S0157	Bergen-Paterson	TVA Serial No. M0247	N/A	2-SNUB-074-5008	N/A	Replaced	No
Support (Snubber) 2-47B452S0157	Fronek (Anchor/Darling)	ADH-3000-780	N/A	2-SNUB-074-5008	N/A	Replacement	No
Support (Snubber) 2-47B452S0240	Anchor/Darling	ADH-3003-418	N/A	2-SNUB-074-5038	N/A	Replaced	No
Support (Snubber) 2-47B452S0240	Fronek (Anchor/Darling)	ADH-3002-784	N/A	2-SNUB-074-5038	N/A	Replacement	No
Support (Snubber) 2-47B408S0227	Anchor/Darling	ADH-3003-421	N/A	2-SNUB-074-5039	N/A	Replaced	No
Support (Snubber) 2-47B408S0227	Fronek (Anchor/Darling)	ADH-3002-786	N/A	2-SNUB-074-5039	N/A	Replacement	No
Support (Snubber) 47B452-1206	Bergen-Paterson	TVA Serial No. M0265	N/A	2-SNUB-074-5010	N/A	Replaced	No
Support (Snubber) 47B452-1206	Bergen-Paterson	TVA Serial No. M0393	N/A	2-SNUB-074-5010	N/A	Replacement	No

See Sheet 3 for continuation of component listing

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

9. Remarks WO 98-012563-000 (2-SNUB-074-5037)

The original snubber (Serial No. ADH-2003-795) was removed and stored in the snubber room as a spare due to seal life expiration. ^{Applicable Manufacturer's Data Reports to be attached} The newly installed snubber (TVA Serial No. M0503) was previously located in Unit 2 as Reactor Recirculation (RECIRC) SSA5 (325) (2-47B408S0073), UNID 2-SNUB-068-5012 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber (TVA Serial No. M0503) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-074-5037.

WO 98-013616-000 (2-SNUB-074-5008)

The original snubber (TVA Serial No. M0247) was removed and stored in the snubber room as a spare due to seal life expiration. ^{Applicable Manufacturer's Data Reports to be attached} The newly installed snubber (Manufacturer Serial Number ADH-3000-780) was purchased under contract 99N2R-244798-000. The new snubber (Manufacturer Serial Number ADH-3000-780) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-074-5008. Fronek purchased Anchor/Darling.

WO 98-013696-000 (2-SNUB-074-5038)

The original snubber (Serial No. ADH-3003-418) was removed and stored in the snubber room as a spare due to seal life expiration. ^{Applicable Manufacturer's Data Reports to be attached} The newly installed snubber (Serial No. ADH-3002-784) was purchased under contract 99N2R-244798-000. The new snubber (Serial No. ADH-3002-784) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-074-5038. Fronek purchased Anchor/Darling.

WO 98-013698-000 (2-SNUB-074-5039)

The original snubber (Serial No. ADH-3003-421) was removed and stored in the snubber room as a spare due to seal life expiration. ^{Applicable Manufacturer's Data Reports to be attached} The newly installed snubber (Serial No. ADH-3002-786) was purchased under contract 99N2R-244798-000. The new snubber (Serial No. ADH-3002-786) was functionally tested by the manufacturer and found acceptable, prior to installation at 2-SNUB-074-5039. Fronek purchased Anchor/Darling.

WO 99-003435-001 (2-SNUB-074-5010)

The original snubber (TVA Serial No. M0265) was removed and stored in the snubber room as a spare. The newly installed snubber (TVA Serial No. M0393) which ^{Applicable Manufacturer's Data Reports to be attached} was previously located in Unit 2 as Residual Heat Removal (RHR) 2-47B452R0051 North UNID 2-SNUB-074-5011 which had been replaced by 2-SI-4.6.H-2B and kept as a spare. The new snubber was rebuilt in accordance with MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly, and the snubber was functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-074-5010.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Date June 14, 1999
1101 Market Street
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 3 of 3
P. O. Box 2000, Decatur, AL 35609-2000 Unit 2
Name
Address
3. Work Performed by TVA-BFN Work Orders (WO) 98-011555-000, 98-012561-000,
P. O. Box 2000, Decatur, AL 35609-2000 98-012563-000, 98-013616-000, 98-013696-000,
98-013698-000, 99-003435-001, 99-003436-017,
99-003436-034, 99-003437-001
Repair Organization P.O. No., Job No., etc.
Type Code Symbol Stamp N/A
4. Identification of System: System 074, Residual Heat Removal System Authorization No. N/A
Expiration Date N/A
5. (a) Applicable Construction Code USAS B31.1.0 19 67* Edition, N/A Addenda, N/A Code Case
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B452R0051 South	Bergen-Paterson	TVA Serial No. M0465	N/A	2-SNUB-074-5012	N/A	Replaced	No
Support (Snubber) 2-47B452R0051 South	Bergen-Paterson	TVA Serial No. M0465	N/A	2-SNUB-074-5012	N/A	Replacement	No
Support (Snubber) RHR R-158 South	Bergen-Paterson	TVA Serial No. M0358	N/A	2-SNUB-074-5031	N/A	Replaced	No
Support (Snubber) RHR R-158 South	Fronek (Anchor/Darling)	ADH-300-2047	N/A	2-SNUB-074-5031	N/A	Replacement	No
Support (Snubber) 2-47B452S0251	Bergen-Paterson	TVA Serial No. M0164	N/A	2-SNUB-074-5005	N/A	Replaced	No
Support (Snubber) 2-47B452S0251	Anchor/Darling	ADH-2003-795	N/A	2-SNUB-074-5005	N/A	Replacement	No

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

9. Remarks WO 99-003436-017 (2-SNUB-074-5012)

The original snubber (TVA Serial No. M0465) was rebuilt using the main cylinder, piston and piston rod from spare snubber

Applicable Manufacturer's Data Reports to be attached

M0284, which had been kept in the snubber room after removal location Residual Heat Removal R041W, UNID 2-SNUB-074-5010.

The reinstalled snubber (TVA Serial No. M0465) was functionally tested and found acceptable, prior to installation 2-SNUB-074-5012.

WO 99-003436-034 (2-SNUB-074-5031)

The original snubber (TVA Serial No. M0358) was removed and stored in the snubber room as a spare due to seal life

Applicable Manufacturer's Data Reports to be attached

expiration. The newly installed snubber (Serial No. ADH-300-2047) was purchased under contract P-99N2D-252831-000.

The new snubber (Serial No. ADH-300-2047) was functionally tested by the manufacturer and found acceptable, prior to installation

at 2-SNUB-074-5031. Fronek purchased Anchor/Darling.

WO 99-003437-001 (2-SNUB-074-5005)

The original snubber (TVA Serial No. M0164) was removed and stored in the snubber room as a spare.

Applicable Manufacturer's Data Reports to be attached

The newly installed snubber (Serial ADH-2003-795) was previously located in Unit 2 as Residual Heat Removal 2-47B452S0235

UNID 2-SNUB-074-5037. The newly installed snubber (Serial ADH-2003-795) was rebuilt per MPI-0-000-SNB002, Hydraulic

Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B. Functional Testing of

Bergen-Paterson Hydraulic Snubbers, prior to installation at 2-SNUB-074-5005.

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 2, 1999

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order 98-012557-000
Repair Organization P.O. No., Job No., etc

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 075, Core Spray (CS) System

5. (a) Applicable Construction Code USAS B31.1:0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Support (Snubber) 2-47B458S0004	Bergen-Paterson	TVA Serial No. M0044	N/A	2-SNUB-075-5021	N/A	Replaced	No
Support (Snubber) 2-47B458S0004	Bergen-Paterson	TVA Serial No. M0448	N/A	2-SNUB-075-5021	N/A	Replacement	No

7. Description of Work Replaced snubber 2-SNUB-075-5021 with like-for-like; this snubber is an ASME Code Class 1 equivalent component.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure **** See Remarks**
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91602 and Design Criteria BFN-50-7075 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks The original snubber (TVA Serial No. M0044) was removed and stored in the snubber room as a spare due to seal life expiration. The newly installed snubber (TVA Serial No. M0448) was previously located in Unit 3 as Feedwater (FW) 3-47B415-52 (SSB1) UNID 3-SNUB-003-5024 which had been replaced by MMI-45 and kept as a spare. The new snubber (TVA Serial No. M0448) was rebuilt per MPI-0-000-SNB002, Hydraulic Shock and Sway Arrestor Bergen-Paterson Unit Disassembly and Reassembly, and functionally tested per 2-SI-4.6.H-2B, Functional Testing of Bergen-Paterson Snubbers, prior to installation at 2-SNUB-075-5021.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this Replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed Stephen C. Willard, System Engineer
Owner or Owner's Designer, Title

Date 6-8, 19 99

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBI&I of HARTFORD, CT have inspected the components described in this Owner's Report during the period 2/24/99 to 4/14/99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd
Inspector's Signature

Commissions

TN 3135 IB SNA
National Board, State, Province, and Endorsements

Date June 11 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date May 17, 1999

Sheet 1 of 2

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Design Change Notice (DCN) S18883A
 Work Order (WO) 98-015377-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 085, Control Rod Drive System

5. (a) Applicable Construction Code ASME Section III 19 74 Edition, Winter 1975 Addenda, N207 1361-2 Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Control Rod Drive Mechanism 06-31	General Electric Nuclear Energy	A4813	N/A	2-CRDM-085-06-31 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 10-19	General Electric Nuclear Energy	A2312	N/A	2-CRDM-085-10-19 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 10-51	General Electric Nuclear Energy	A3670	N/A	2-CRDM-085-10-51 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 14-27	General Electric Nuclear Energy	A4689	N/A	2-CRDM-085-14-27 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 14-35	General Electric Nuclear Energy	A3742	N/A	2-CRDM-085-14-35 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 18-27	General Electric Nuclear Energy	A4565	N/A	2-CRDM-085-18-27 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 18-31	General Electric Nuclear Energy	A3218	N/A	2-CRDM-085-18-31 BWR/6	1996	Replacement	Yes

See Sheet 2 for continuation of component listing

7. Description of Work Replaced 28 BWR/4 Control Rod Drives (CRD) with BWR/6 CRDs.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

FORM NIS-2 (Back)

9. Remarks Replaced 28 BWR/4 Control Rod Drives (CRD) with BWR/6 CRDs. The N-2 data reports are attached.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed [Signature] System Engineer
Owner or Owner's Designee, Title

Date 6-29 .1999

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSB&I of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4-7-99 to 5-9-99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]
Inspector's Signature

Commissions TN3135 TR5NA
National Board, State, Province, and Endorsements

Date June 30 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS SUPPLEMENTAL SHEET

1. Owner Tennessee Valley Authority (TVA) Date May 17, 1999
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Sheet 2 of 2
Name
P. O. Box 2000, Decatur, AL 35609-2000 Unit 2
Address
 Design Change Notice (DCN) S18883A
 Work Order (WO) 98-015377-000
Repair Organization P.O. No., Job No., etc
 Type Code Symbol Stamp N/A
3. Work Performed by TVA-BFN Authorization No. N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000 Expiration Date N/A
Address
4. Identification of System System 085 Control Rod Drive System
5. (a) Applicable Construction Code ASME Section III, 1974 Edition, Winter 1975 Addenda, N207 1361-2 Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89
6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Control Rod Drive Mechanism 18-43	General Electric Nuclear Energy	A4368	N/A	2-CRDM-085-18-43 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 22-31	General Electric Nuclear Energy	A3236	N/A	2-CRDM-085-22-31 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 22-47	General Electric Nuclear Energy	A2181	N/A	2-CRDM-085-22-47 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 26-19	General Electric Nuclear Energy	A4251	N/A	2-CRDM-085-26-19 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 26-55	General Electric Nuclear Energy	A2285	N/A	2-CRDM-085-26-55 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 30-43	General Electric Nuclear Energy	A3610	N/A	2-CRDM-085-30-43 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 30-47	General Electric Nuclear Energy	A4826	N/A	2-CRDM-085-30-47 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 34-03	General Electric Nuclear Energy	A3878	N/A	2-CRDM-085-34-03 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 34-11	General Electric Nuclear Energy	A3899	N/A	2-CRDM-085-34-11 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 34-43	General Electric Nuclear Energy	A3589	N/A	2-CRDM-085-34-43 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 38-03	General Electric Nuclear Energy	A4823	N/A	2-CRDM-085-38-03 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 38-19	General Electric Nuclear Energy	9184	N/A	2-CRDM-085-38-19 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 38-51	General Electric Nuclear Energy	A5601	N/A	2-CRDM-085-38-51 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 42-59	General Electric Nuclear Energy	A5042	N/A	2-CRDM-085-42-59 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 46-35	General Electric Nuclear Energy	A4298	N/A	2-CRDM-085-46-35 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 46-55	General Electric Nuclear Energy	A3758	N/A	2-CRDM-085-46-55 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 50-27	General Electric Nuclear Energy	A5323	N/A	2-CRDM-085-50-27 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 50-43	General Electric Nuclear Energy	A4809	N/A	2-CRDM-085-50-43 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 54-15	General Electric Nuclear Energy	A4722	N/A	2-CRDM-085-54-15 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 54-35	General Electric Nuclear Energy	A5524	N/A	2-CRDM-085-54-35 BWR/6	1996	Replacement	Yes
Control Rod Drive Mechanism 58-43	General Electric Nuclear Energy	A4156	N/A	2-CRDM-085-58-43 BWR/6	1996	Replacement	Yes

FORM NIS-2, SUPPLEMENTAL SHEET (Back)

Remarks See back of sheet 1.

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002485

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4813 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS Standard part for use with Reactor Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design, Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By P. Shetty
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99, Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B. N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/9, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9, 1996
Date

James P. Evers
Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)



FORM N-2 (back)

002486

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7 Jacket Closure: _____
(Describe as open and used, bar, etc. if bar give dimensions, if bolts, describe or sketch)

8 Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating, Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner channels of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13 Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14 Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17 Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18 Supports Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

¹ If Prolonged Head Trapped
² List other external or internal pressure with corresponding temperature when applicable



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)

002487

3901 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA DECATUR, AL 35609-2000

(Name and Address of E Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/M of Part : A4813 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

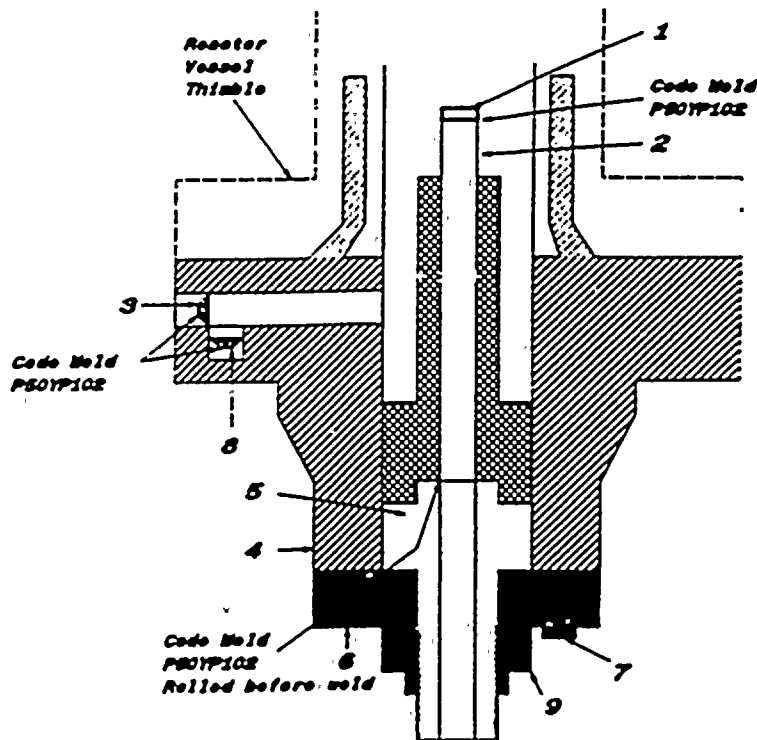
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.

(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C3311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C3934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





002513

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of NPT Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A4823 Mat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 763E524G008 Rev 9 Desg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1381-2 Class 1
- 3. REMARKS. Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed _____ By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MQ18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/22, 1996, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9, 1996 [Signature] NC 1231, Ohio, WC 3688 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(9/96)



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

001475

- 1 Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of R Certificate Holder for completed nuclear component)
- 2 Identification - Certificate Holder's S/N of Part : A2312 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3 REMARKS Standard part for use with Reactor. Hydrostatically tested at 1825 psi min
 (Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By [Signature]
 (NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
 Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
 Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/20, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 [Signature] NC 1231, Ohio, WC 3686 PA
 Date Inspector's Signature National Board, State, Province And No

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

(07/90)



FORM M-2 (back)

001476

Items 4-8 Incl. to be completed for single well vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Spec Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as type and seal, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary, Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating, Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in Thickness _____ inches or gage. Number _____ Type _____
(Dr. or U)

Items 11 - 14 incl. to be completed for inner diameters of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) _____ Number _____ Dia. or Size _____ Type _____ Material _____ Thickness _____ Reinforcement Material _____ How Attached _____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Manholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat Treated
 2 List other internal or external pressure with corresponding temperature when applicable



001778

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A3870 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 78DB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
- 3. REMARKS Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A8253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A8254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/8, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 10/8, 1996 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3698 PA

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".



FORM N-2 (back)

601773

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open and used, hot, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____ ft-lb
 Charpy Impact _____

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner channels, jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____ ft-lb
 Charpy Impact _____

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Vent, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat Treated.
 2 List other internal or external pressure with corresponding temperature when applicable.

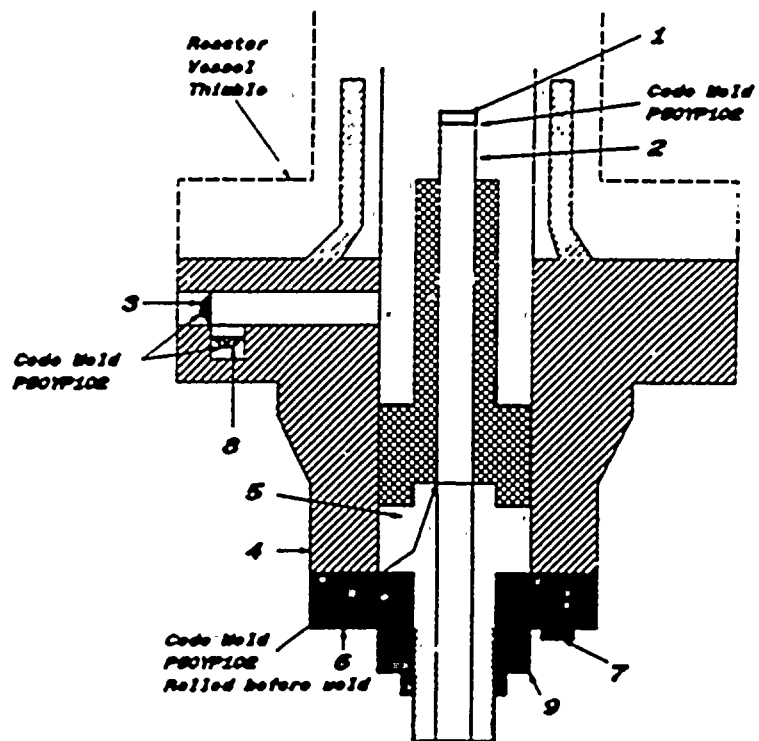


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) 001780
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of E Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3670 Part Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207.1361-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 91D0610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA182 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





G02256

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A4689 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 738E534G008 Rev 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3. REMARKS. Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed _____ By Calvin...
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/17, 1996, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III. By signing this certificate, neither the inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 10/9, 1996 Inspector's Signature James P. Evers National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/96)



002257

FORM M-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Code & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) _____
 (b) _____
 If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.R. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open end weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 psi at _____ 575 °F at temp of _____ °F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Code & Spec. No.) (Subject to pressure) (Welded, Bolted)
 Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Code & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) Top, bottom, ends: _____
 (b) Channel _____
 If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ °F at temp of _____ °F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) Number Dia. or Size Type Material Thickness Reinforcement Material How Attached

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached

17. Inspection Manholes, No. _____ Size _____ Location _____
 Openings: Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1. If Punctured Head Tested
 2. List other internal or external pressure with concurrent temperature when applicable



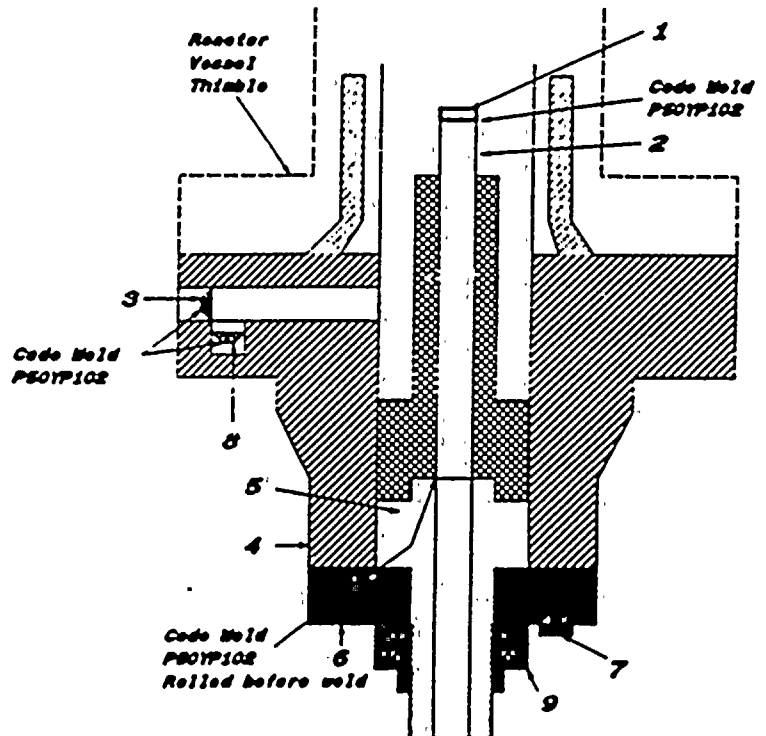
602258

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4689 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Des. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psf. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 156A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 9180610P001 (710E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 11485122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7081P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





601806

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
- 2 Identification - Certificate Holder's S/N of Part : A3742 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1975, Addenda Date W75, Case No. N207 1361-2, Class 1
- 3 REMARKS, Standard part for use with Reactor. Hydrostatically tested at 1825 psi min
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By U. Baggett
(NPT Certificate Holder) (JC QA Representative)

Certificate of Authorization Expires, 8/16/99 Certification of Authorization No. : NPT N - 1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MA018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 770, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 Shirone P. Evers
Date Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)



FORM N-2 (back)

001807

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____

(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ (Material, Spec. No., T.S. Size Number) Other fastening _____ (Describe or attach sketch)

7. Jacket Closure: _____ (Describe as pipe and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____ ft-lb
Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner channels of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____

(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____ (Describe or attach sketch)

Drop Weight _____ ft-lb
Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) _____ Number _____ Dia. or Size _____ Type _____ Material _____ Thickness _____ Reinforcement Material _____ How Attached _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Manholes, No. _____ Size _____ Location _____
Openings: Handholes, No. _____ Size _____ Location _____
Threaded, No. _____ Size _____ Location _____

18. Supports. Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat Treated.
2 - List either internal or external pressure with corresponding temperature when applicable.

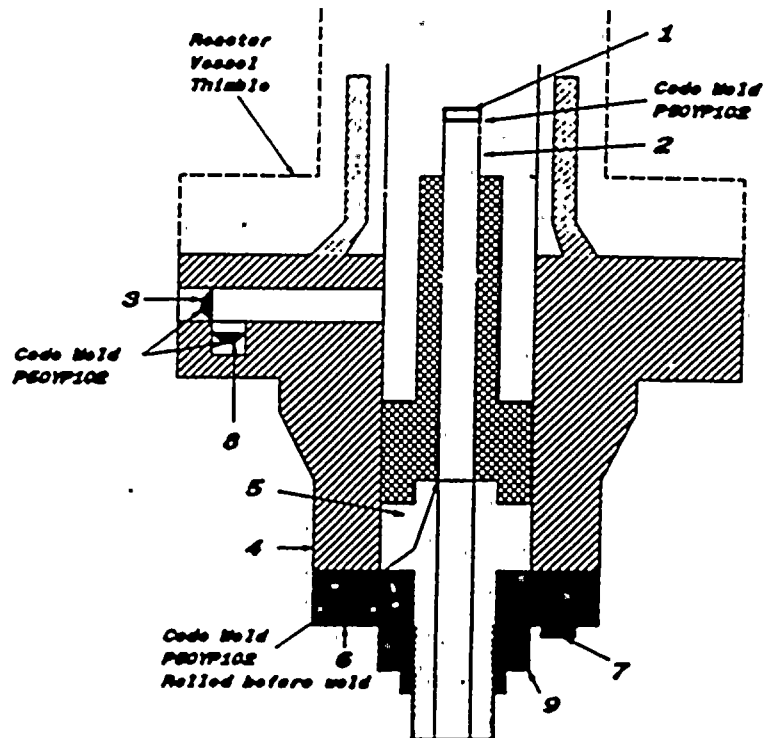


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) 001808
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3742 Part Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA152 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 158A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





00228

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35608-2000
 (Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4565 Part Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min
 (Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress report)

Date: 10/08/96 Signed GE By [Signature]
(NPT Certificate Holder) (SC GA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. NPT N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MO18646

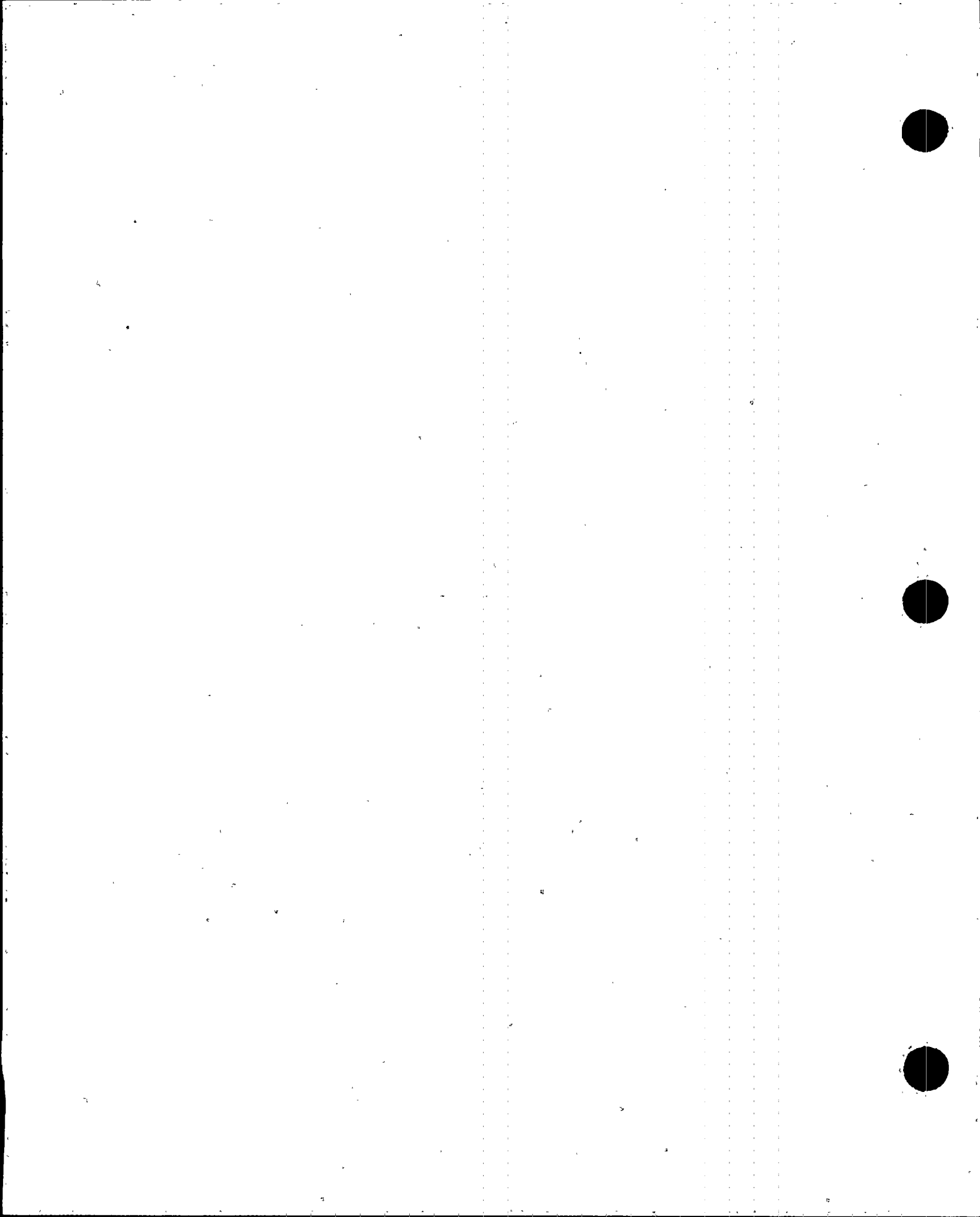
Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina, have inspected the part of a pressure vessel described in this Partial Data Report on 5/17, 1996, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9, 1996 [Signature] NC 1231, Ohio, W.C. 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/88)



FORM M-2 (back)

002229

Items 4-8 incl. to be completed for single wall vessels, jackets, vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. or Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7 Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

8 Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F
 Drop Weight _____ Charpy Impact _____ ft-lb

Items 9 and 10 to be completed for tube sections

9 Tube Sheets: Stationary, Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating, Material _____ Dia. _____ Thickness _____ in. Attachment _____

10 Tubes: Material _____ C.O. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner structures of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. or Range Specified)

12 Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13 heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14 Design pressure ² _____ psi at _____ ° F at temp of _____ ° F
 Drop Weight _____ Charpy Impact _____ ft-lb

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Vent, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17 Inspection: Manholes, No. _____ Size _____ Location _____
 Openings: Manholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18 Supports Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Pressure Head-Treated
 2 List other internal or external pressure with pertinent temperature when applicable

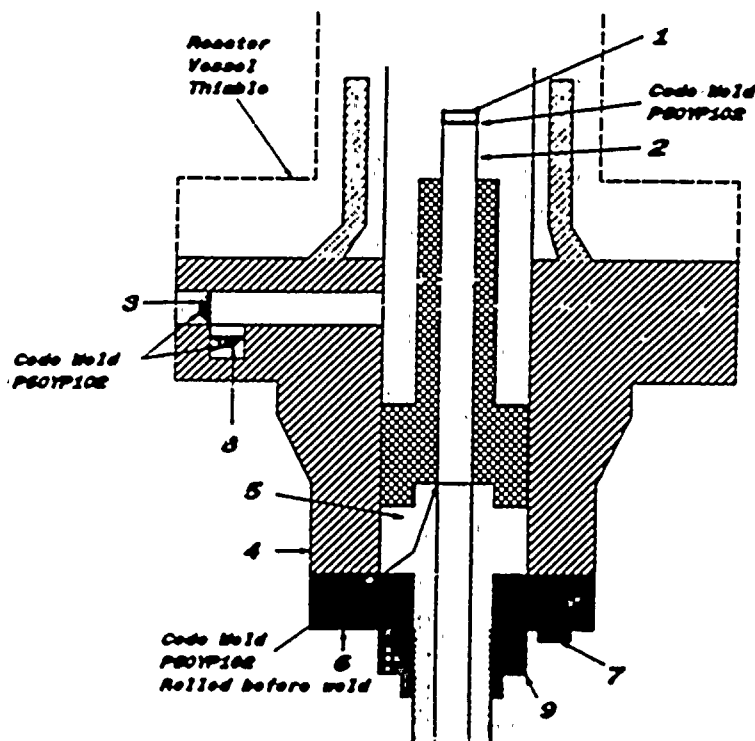


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 002230
1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35809-2000
(Name and Address of N Certificate Holder for completed nuclear component)
 2. Identification - Certificate Holder's S/N of Part : A4565 Mat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive Model # 78DB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
 3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi/min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 918D610P001 (718E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA183 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C8934P001
XM - 18 SA479
1.30" thick x 2.62" dia.





001557

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A3218 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Des. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By C. Bryant
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/2, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 James P. Egan NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)



FORM M-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia _____ ft. _____ in. Length _____ ft. _____
(Wind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material: _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe an edge and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Wind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.C. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner _____ of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Wind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) _____ Number _____ Dia. or Size _____ Type _____ Material _____ Thickness _____ Reinforcement Material _____ How Attached _____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1. If Pressure Heat Treated.
 2. List other internal or external pressure with operating temperature when applicable

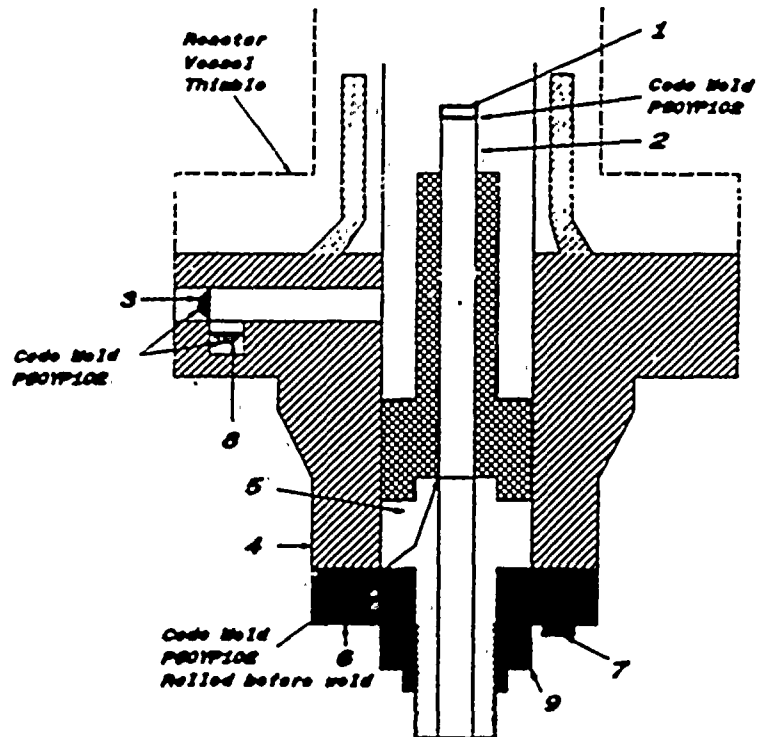


FORM N-21NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) 001559
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35009-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3218 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 788E534G008 Rev. 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 78DB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207.1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B0274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 158A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114BS122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





CC2116

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3801 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4368 Netl Bd. No. N/A
- (a) Constructed According to Drawing No: 768E514G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3 REMARKS. Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 8/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MQ18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/23/96 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 [Signature] NC 1231, Ohio WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/96)



FORM M-2 (back)

602117

Items 4-8 incl to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4 Shell Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) _____
 (b) _____
 If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.B. Size Number) (Describe or attach sketch)

7. Jacket Closure. _____
(Describe as open end weld, bar, etc. If bar give dimensions, if bolts, describe or attach)
 Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9 Tube Sheets. Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
 Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner members of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Range Specified)

12 Seams Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13 Heads. (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) Top, bottom, ends _____
 (b) Channel _____
 If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

14 Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles:	Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
	_____	_____	_____	_____	_____	_____	_____	_____
	_____	_____	_____	_____	_____	_____	_____	_____
	_____	_____	_____	_____	_____	_____	_____	_____

17 Inspection Manholes, No _____ Size _____ Location _____
 Openings: Manholes, No. _____ Size _____ Location _____
 Threaded, No _____ Size _____ Location _____

18 Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

¹ If Pressure Head Traced
² List other internal or external pressure with conditions temperature when applicable



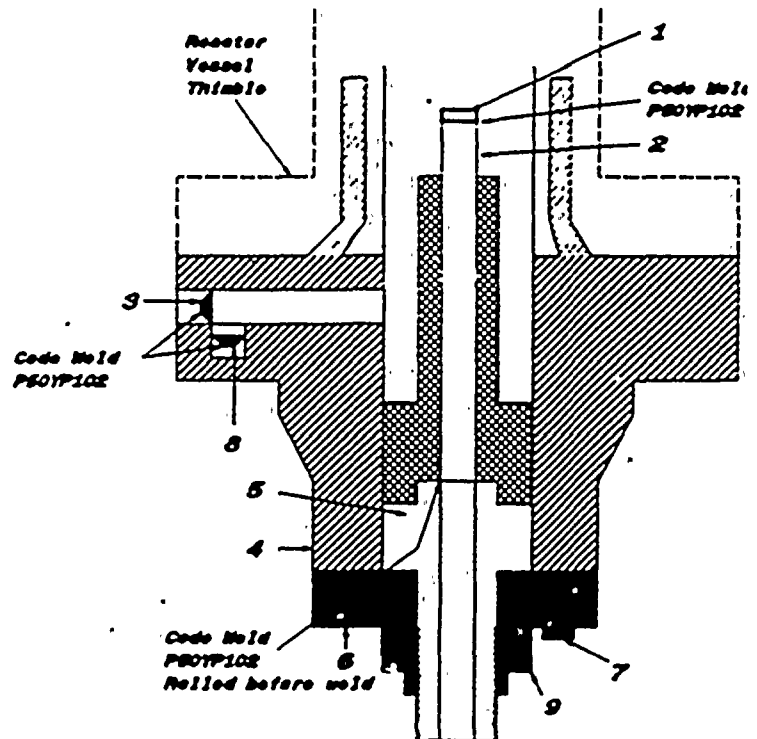
FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 *As required by the Provision of the ASME Code Rules, Section III, Div. I

002118

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR AL 35809-2000
 (Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4389 Net/Ed. No. N/A
 - (a) Constructed According to Drawing No: 708E534G008 Rev 2, Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive Model # 7RDB144EG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Data W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi/min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001.
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137CS311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA183 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137CS834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





001613

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1 Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
- 2 Identification - Certificate Holder's S/N of Part : A3236 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No. 768E534G008 Rev 9 Desg. Prepared by D L Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code Section III, Edition 1974, Addenda Date W75, Case No N207 1361-2 Class 1
- 3 REMARKS Standard part for use with Reactor. Hydrostatically tested at 1825 psi min
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 10/08/96 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California
 Stress analysis report on file at GE Company, San Jose, California
 DC22A6253 Rev 2
 Design specification certified by B.N. Sridhar Prof Eng State Calif Reg. No 18345
 DC22A6254 Rev 1
 Stress analysis report certified by Edward Yoshio Prof Eng. State Calif Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 8/20, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 Date [Signature] Inspector's Signature NC 1231, Ohio, WC 3686 PA National Board, State, Province And No

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)

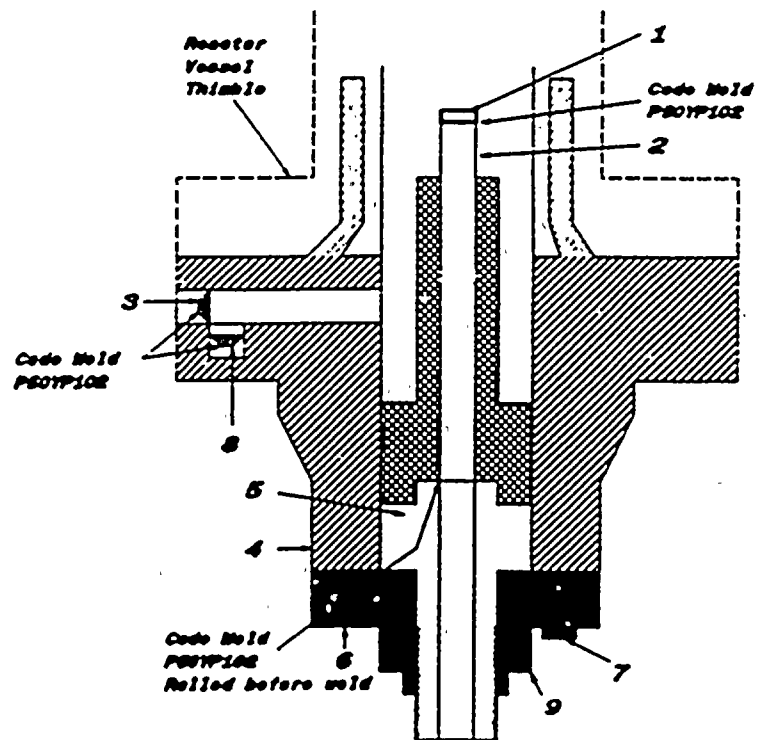


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 *As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) 001614
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35809-2000
 (Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3238 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9, Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Cont'ol Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class I
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B0274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B0313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D0810P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4518P002
SA182 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5034P001
XM - 19 SA479
1.30" thick x 2.62" dia.





001421

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35603-2000
(Name and Address of R Certificate Holder for completed nuclear component)
- 2 Identification - Certificate Holder's S/N of Part : A2181 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3 REMARKS. Standard part for use with Reactor, Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By W. Payne
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California
 Stress analysis report on file at GE Company, San Jose, California
 DC22A6253 Rev. 2
 Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345
 DC22A6254 Rev 1
 Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/30, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
 By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 Date James P. Evers Inspector's Signature NC 1231, Ohio, WC 3688 PA National Board, State, Province And No.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/00)

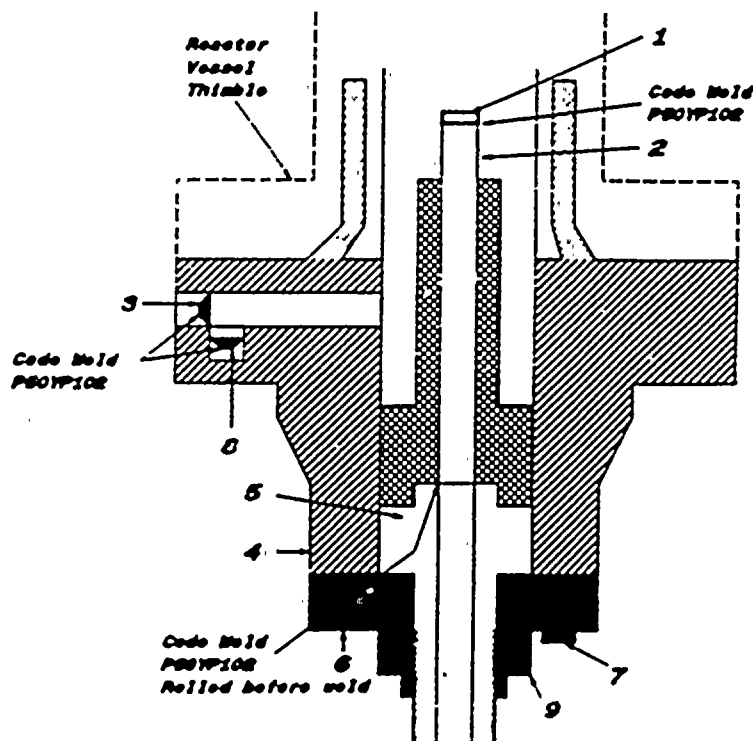


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GENE) 001422
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35902-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A2181 Mat'l Bld. No. N/A
- (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA183 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 18 SA479
1.30" thick x 2.62" dia.





002033

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4251 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 788E534G008 Rev 9 Des. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 78DB144EG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 8/18/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshko Prof. Eng. State Calif. Reg. No. MD18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/17/96, 11/96, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 10/8-1996 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC:3886 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(9/96)



FORM N-2 (back)

002034

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material: _____ T.S. _____ (b) Material _____ T.S. _____
 Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) _____
 (b) _____
 If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)
 Drop Weight _____ ft-lb
 Charpy Impact _____ °F

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ °F at temp of _____ °F

Items 9 and 10 to be completed for tube sections

9. Tube-Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
 Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner cross-section of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material: _____ T.S. _____ (b) Material _____ T.S. _____
 Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) Top, bottom, ends _____
 (b) Channel _____
 If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
 Drop Weight _____ ft-lb
 Charpy Impact _____ °F

14. Design pressure ² _____ psi at _____ °F at temp of _____ °F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) Number Dia. or Size Type Material Thickness Reinforcement Material How Attached

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Manholes, No. _____ Size _____ Location _____
 Openings: Manholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports Skirt _____ Lugs: _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat Treated
 2 List other internal or external pressure with corresponding temperature when applicable



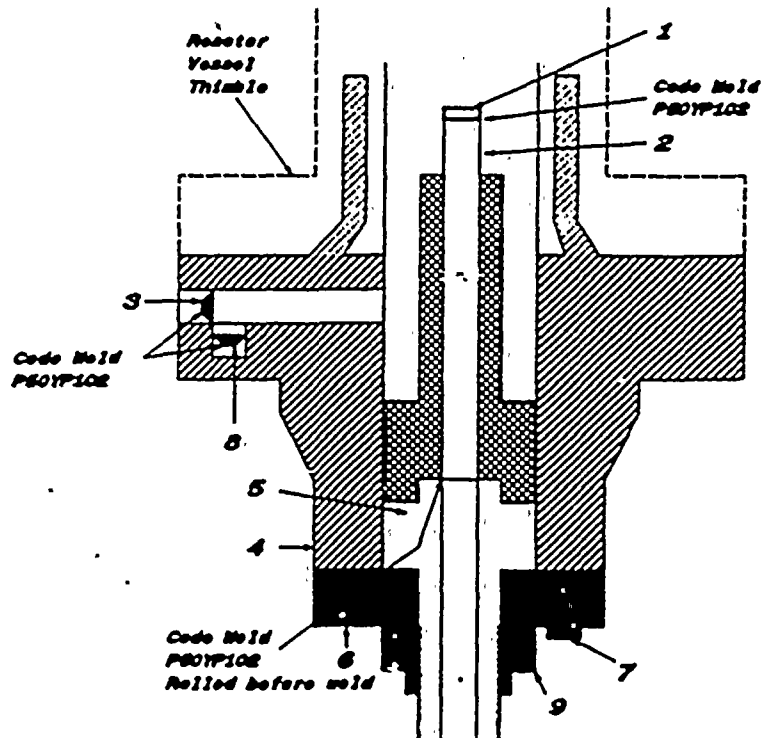
FORM N-3 NPT-CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

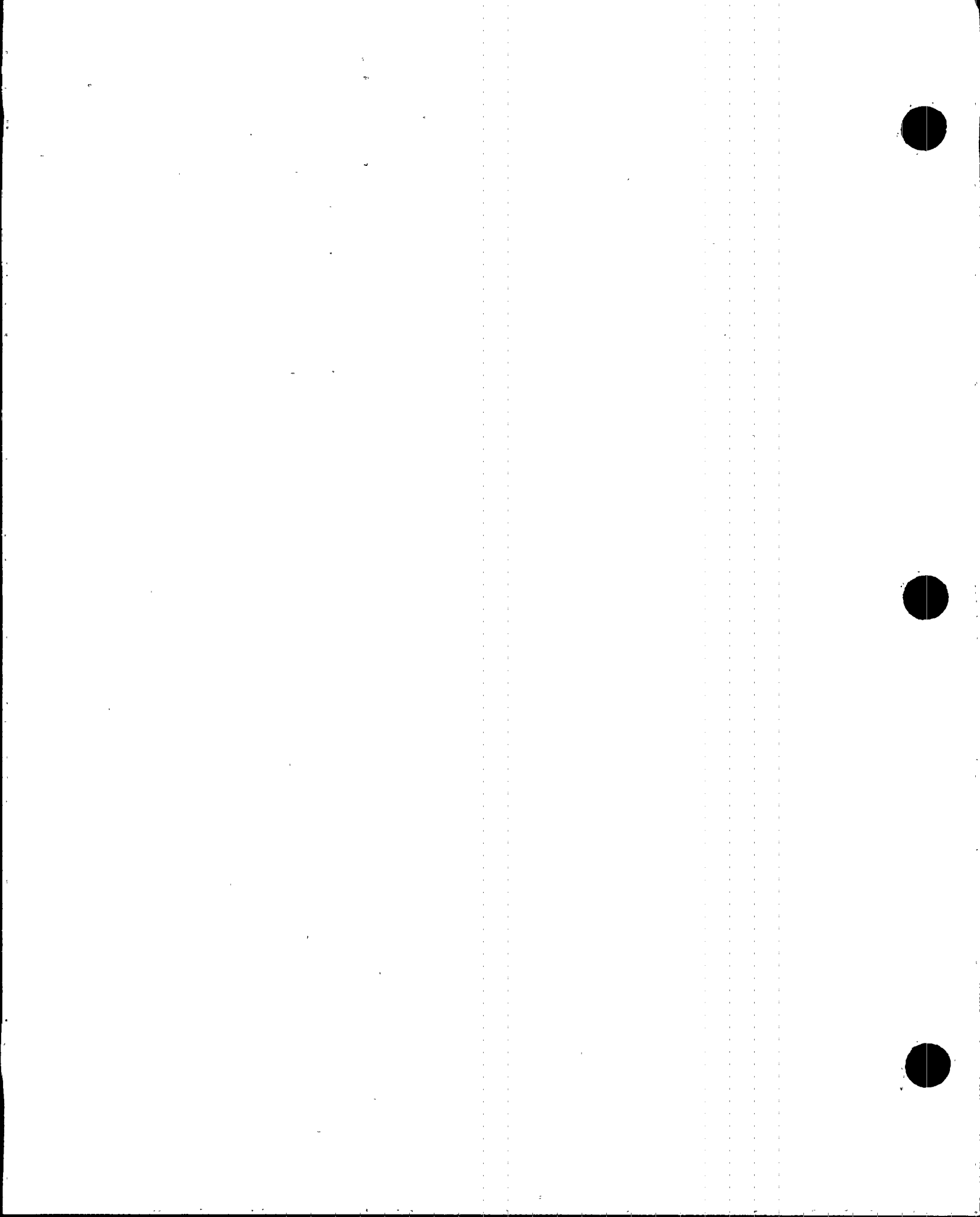
002035

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35802-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4251 Mat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 186B0274P001
 SA182 - F316
 3/8" thick x 1 1/16" OD
2. Indicator Tube 186B0313P001
 SA312 - TP316
 3/4" sch 40 - seamless pipe
 0.113" wall thickness
 1.065" max. dia.
3. Plug 159A1176P001
 SA182 - F304
 1/4" thick x 0.812" OD
4. Flange 9100610P001 (719E474)
 SA182 - F304
 3.37" thick x 9 5/8" OD
5. Base 137CS311P001
 SA182 - F304
 7/8" thick x 2.875" dia.
6. Ring Flange 114BS122P002, P003
 137CB151P001, P002
 SA182 - F304
 1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4518P002
 SA183 - B6
 6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
 SA182 - F304
 0.36" thick x 1.307" dia.
9. Nut 137CS804P001
 XM - 19 SA479
 1.30" thick x 2.62" dia.





FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)

001448

3901 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA DECATUR, AL 35609-2000

(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A2285 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 2 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive Model # 7RDB144EG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min.

(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 10/08/96

Signed GE-NE By C. Payett
(NPT Certificate Holder) (SC GA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/8, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 James P. Evans
Date Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/96)



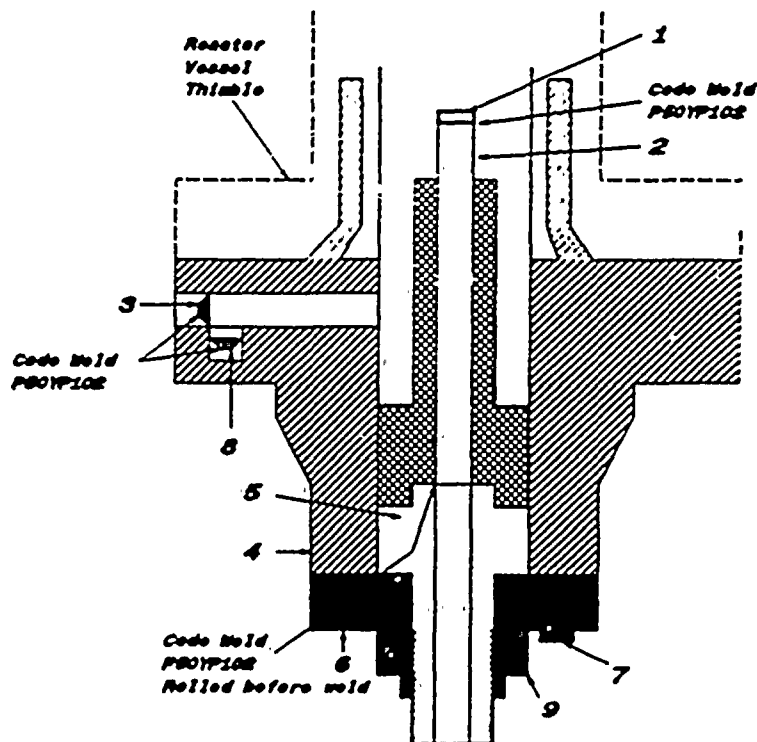
FORM N-2 MPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

001449

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of MPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of E Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A2285 Matl Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1381-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA183 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





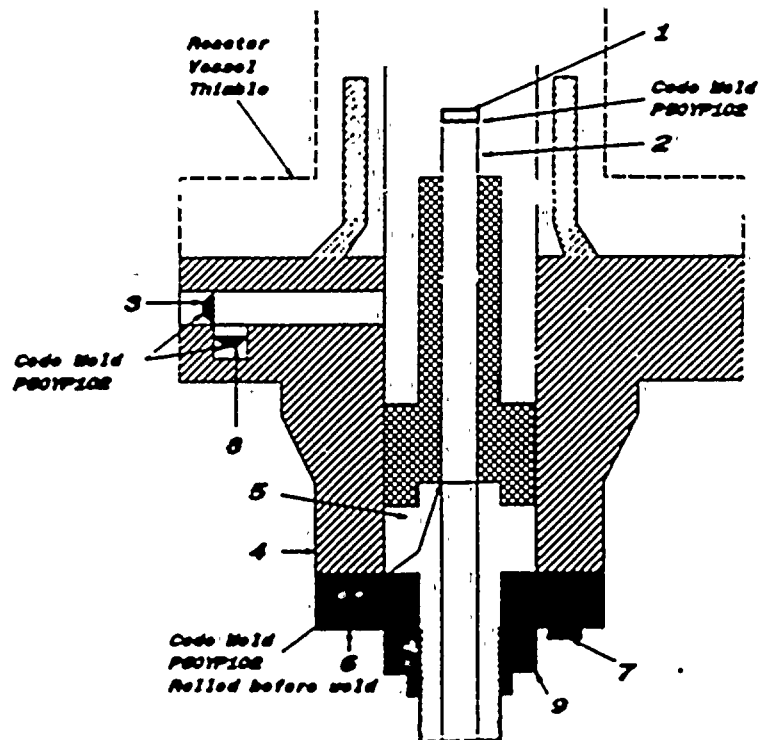
FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

001477

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GENE)
3801 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate holder's S/N of Part : A2312 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive Model # 78DB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 156A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 8190610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA183 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5834P001
JOM - J9 SA479
1.30" thick x 2.62" dia.





001750

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A2810 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207.1361-2, Class 1
3. REMARKS. Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/98 Signed GE-NE By [Signature]
(NPT Certificate Holder) (NPT Certificate Holder)

Certificate of Authorization Expires: 8/18/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 3/23, 1996, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 [Signature]
Date Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/98)



001751

FORM M-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) _____
 (b) _____
 If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Spec Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)
 Drop Weight _____ ft-lb
 Charpy Impact _____

8. Design pressure ² _____ 1250 _____ psi at _____ 525 _____ °F at temp of _____ °F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
 Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner shells of jacket vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) Top, bottom, ends _____
 (b) Channel _____
 If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
 Drop Weight _____ ft-lb
 Charpy Impact _____

14. Design pressure ² _____ psi at _____ °F at temp of _____ °F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain):	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Method	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1. If Positively Heat Treated.
 2. List other normal or external pressure with coincident temperature when applicable.



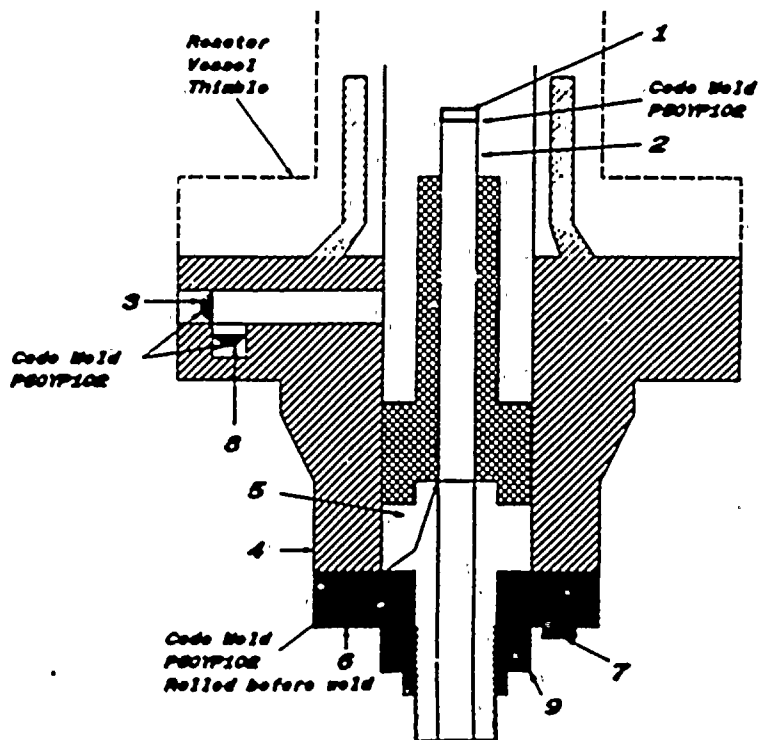
FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

601753

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
- 2: Identification - Certificate Holder's S/N of Part : A3610 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 788E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 78DB144FG005
 - (c) Applicable ASME Code: Section III, Edition: 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1175P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Beam 137CS311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137CS934P001
XM - 19 SA479
1.307" thick x 2.62" dia.





002541

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4826 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed _____ By *Richard*
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 6/9, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9, 1996 *Jerome P. Ewe* NC 1231, Ohio, WG 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/96)



FORM M-2 (back)

Items 4-8 Incl., to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) _____
 (b) _____
 If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or sketch sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
 Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) Top, bottom, ends _____
 (b) Channel _____
 If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or sketch sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

14 Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Vent, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18 Supports. Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Pressure Heat-Treated.
 2 List other internal or external pressure with corresponding temperature when applicable

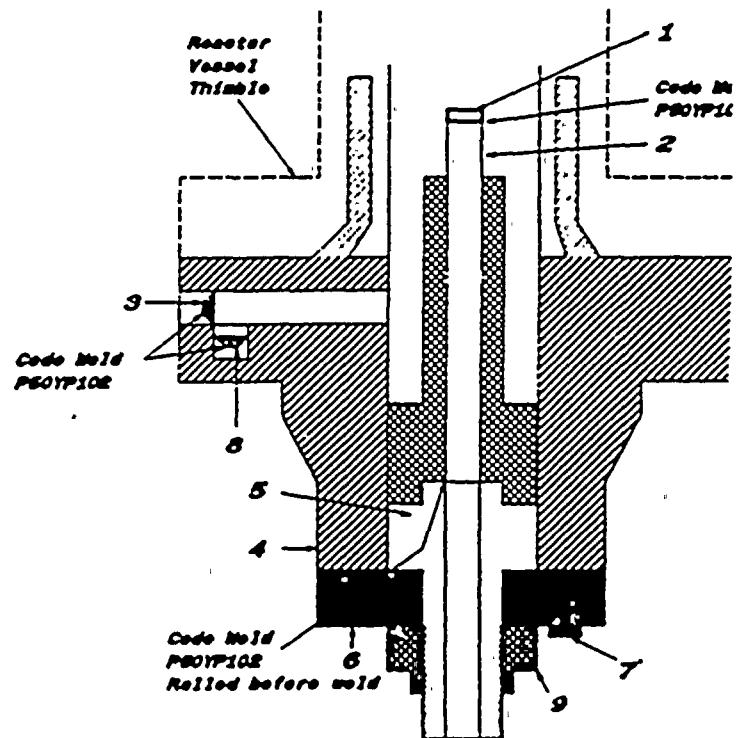


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) 002543
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4826 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1381-2, Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psf. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 158A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 918D810P001 (719E474)
SA182 - F304
3.37" thick x 8 5/8" OD
5. Base 137C3311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C3934P001
XM - 19 SA179
1.30" thick x 2.62" dia.





001866

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A387B Part Id. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 2 Desg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1381-2 Class 1
- 3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/98 Signed GE+ By U. Bryant
(NPT Certificate Holder) (IC QA Representative)

Certificate of Authorization Expires: 8/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MQ18846

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/25, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III. By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 James P. Evers NC 1231, Ohio WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

(10/90)



00-1867

FORM M-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____

(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ (Material, Spec. No., T.S. Size Number) Other fastening _____ (Describe or attach sketch)

7. Jacket Closure: _____ (Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

Drop Weight _____ ft-lb
Charpy Impact _____

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____ (Welded, Bolted)
Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____ (Subject to pressure)

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____ (St. or U)

Items 11 - 14 incl. to be completed for inner jackets of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____

(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____ (Describe or attach sketch)

Drop Weight _____ ft-lb
Charpy Impact _____

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Nozzles: Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
Manholes, No. _____ Size _____ Location _____
Threaded, No. _____ Size _____ Location _____

18. Supports Skirt _____ (Yes or No) Lugs _____ (Number) Legs _____ (Number) Other _____ (Describe) Attached _____ (Where & How)

1. If Phosphor Bronze Treated
2. List other internal or external pressure with corresponding temperature when applicable

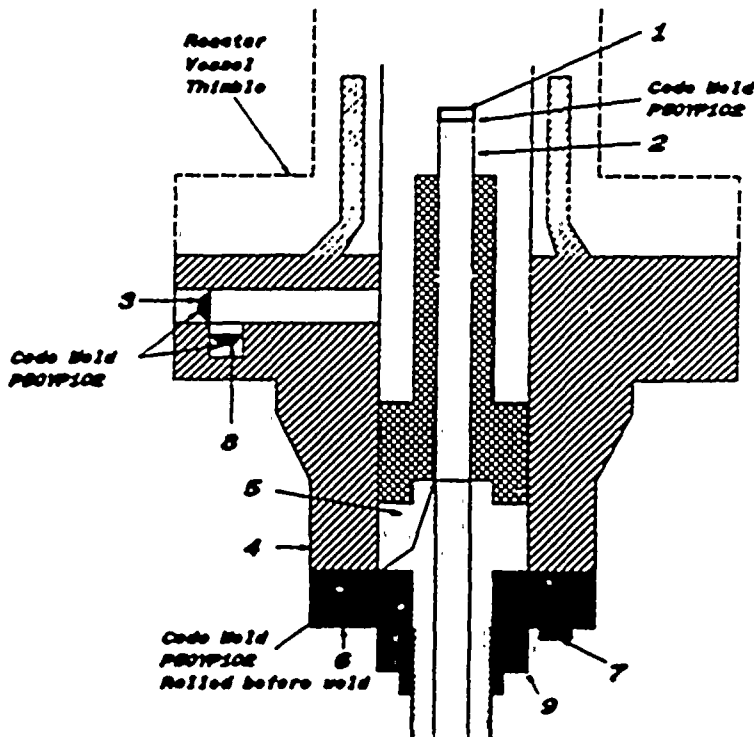


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 *AS Required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) **001868**
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35600-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3878 Net/Bd. No. N/A
- (a) Constructed According to Drawing No: 708F534G008 Rev. 2 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B0274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B0313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919O610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C3311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA182 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C3834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





001894

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3899 Part Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class I
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By CS Bryant
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 8/18/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Snither Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018648

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/8/96 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8/1996 James P. Evers NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/96)



Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe an edge and weld, bar, etc. if bar give dimensions, if both, describe or sketch)

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F
 Drop Weight Charpy Impact _____ ft-lb

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Br. or U)

Items 11 - 14 incl. to be completed for inner jackets, jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F
 Drop Weight Charpy Impact _____ ft-lb

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Manholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat-Treated.
 2 Use either internal or external pressure with maximum temperature when applicable

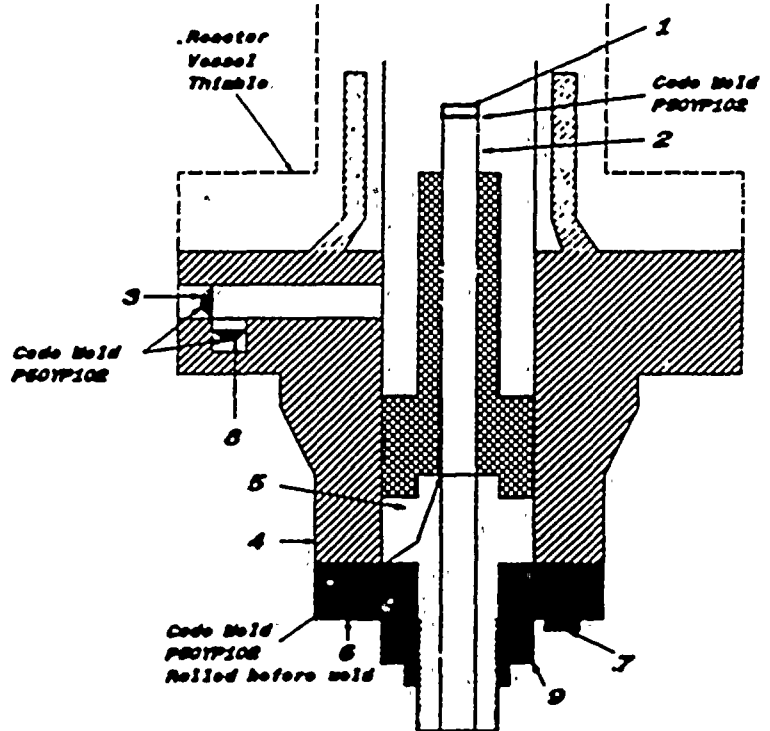


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) **001896**
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3899 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Des. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Pod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psf. min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 918D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137CS311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4518P002
SA183 - B8
6 ea. 1/2" dia. on 4 1/8" ball circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.207" dia.
9. Nut 137CS934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





C01728



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35009-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3589 Net 1 Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Orig. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE+ By W. Baynet
(NPT Certificate Holder) (QC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California
Stress analysis report on file at GE Company, San Jose, California
DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345
DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MQ18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina, and employed by Department of Labor of State of North Carolina, have inspected the part of a pressure vessel described in this Partial Data Report on 10/8, 1996, and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 James P. Lane NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

(07/90)



601724

FORM M-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat-exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open end weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)
 Drop Weight _____ ft-lb
 Charpy Impact _____

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner shells of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Clean)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Manholes, No. _____ Size _____ Location _____
 Openings: Manholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat Treated
 2 List either internal or external pressure with associated temperature when applicable

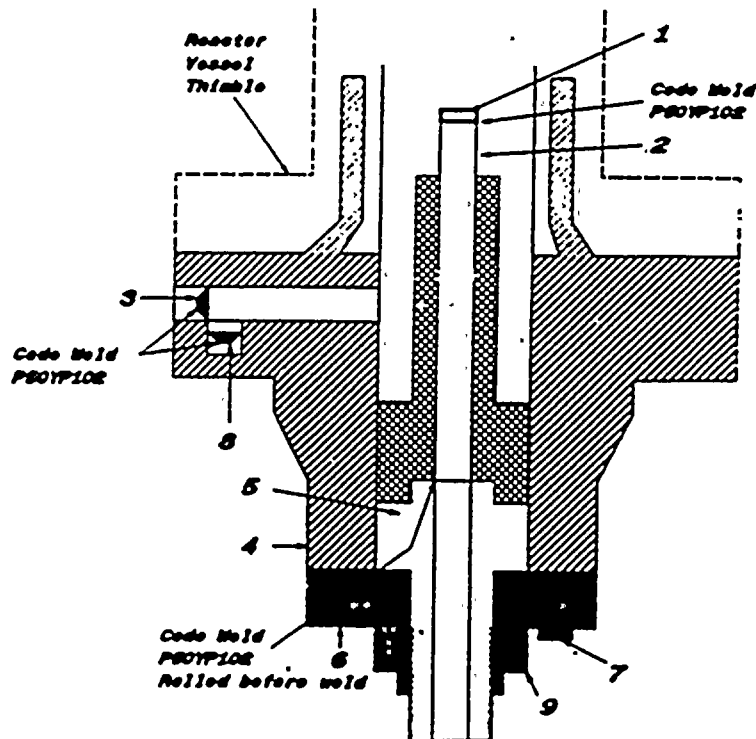


FORM N-2 NPT. CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) 001725
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35809-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3589 Nat'l Bcl. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev. 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 9190610P001 (719E474)
SA182 - F304
3.37" thick x 9.5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4518P002
SA182 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





002513

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35809-2000
(Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4823 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev. 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Design Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 10/08/96 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPT N-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/22, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 10/9, 1996 Inspector's Signature [Signature] National Board, State, Province And No. NC 1231, Ohio, WC 3686 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/96)



002515

FORM N-2 (back)

Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 Incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14 Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat-Treated.
 2 - List other internal or external pressure with coincident temperature when applicable



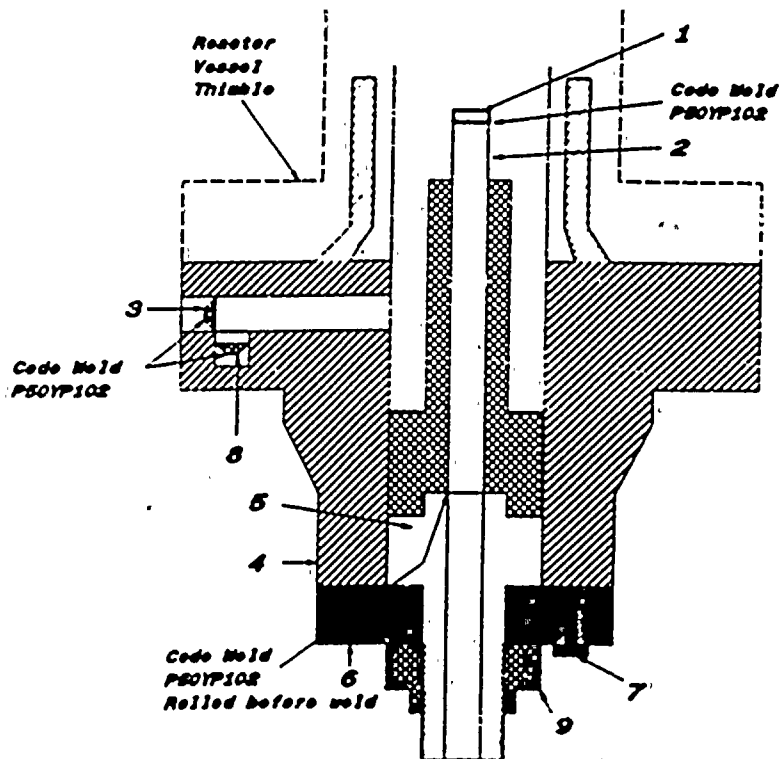
002514

200 FORM NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hsme Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4823 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9, Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 186B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137CS311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117CA516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137CS5834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





001387

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/M of Part : 9184 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 10/08/96 Signed GE-NE By W. Baggett
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California
Stress analysis report on file at GE Company, San Jose, California
DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345
DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 6/28, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 James P. Evers NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/99)

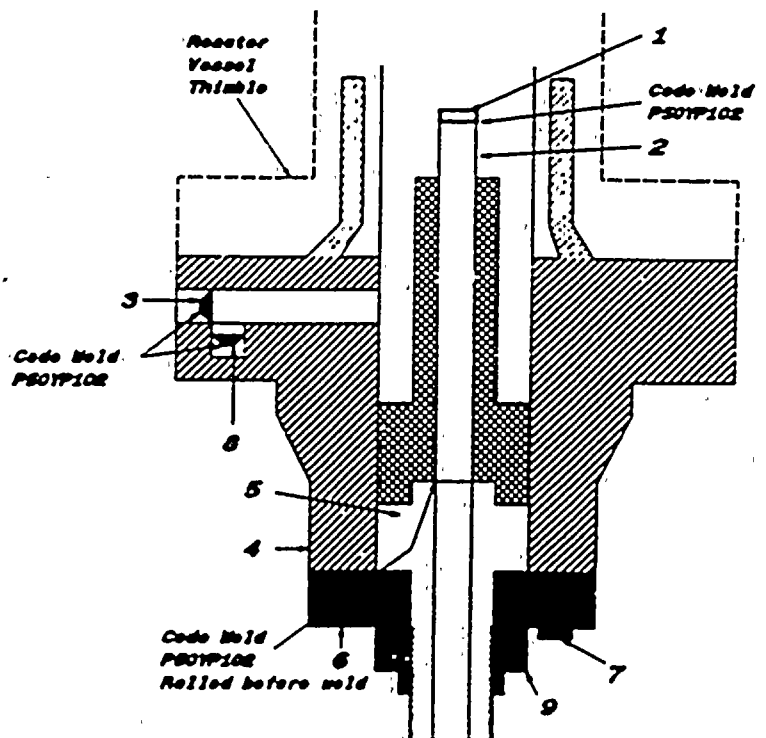


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 *As required by the Provision of the ASME Code Rules, Section III, Div. 1

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Home Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder) 001368
- (b) Manufactured for : TVA DECATUR, AL 35009-2000
 (Name and Address of NPT Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : 9184 Mat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # TRDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 9190810P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA183 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 19 SA479
1.30" thick x 2.62" dia.





002758

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)

3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA DECATUR, AL 35609-2000

(Name and Address of NPT Certificate Holder for completed nuclear component)

2 Identification - Certificate Holder's S/N of Part : A5601 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1

3 REMARKS Standard part for use with Reactor Hydrostatically tested at 1825 psi min.

(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 10/08/96

Signed GE NE

By [Signature]

(Name and Address of Certificate Holder)

(SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina, and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 6/18, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9, 1996
Date

[Signature]
Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)



Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4 Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) _____
 (b) _____
 if removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7 Jacket Closure _____
(Describe as open and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

8 Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
 Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Dr. or U)

Items 11 - 14 incl. to be completed for inner members of jacketed vessels, or channels of heat exchangers.

11. Shell Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12 Seams Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13 Heads (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) Top, bottom, ends _____
 (b) Channel _____
 if removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
 Drop Weight _____
 Charpy Impact _____ ft-lb

14 Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

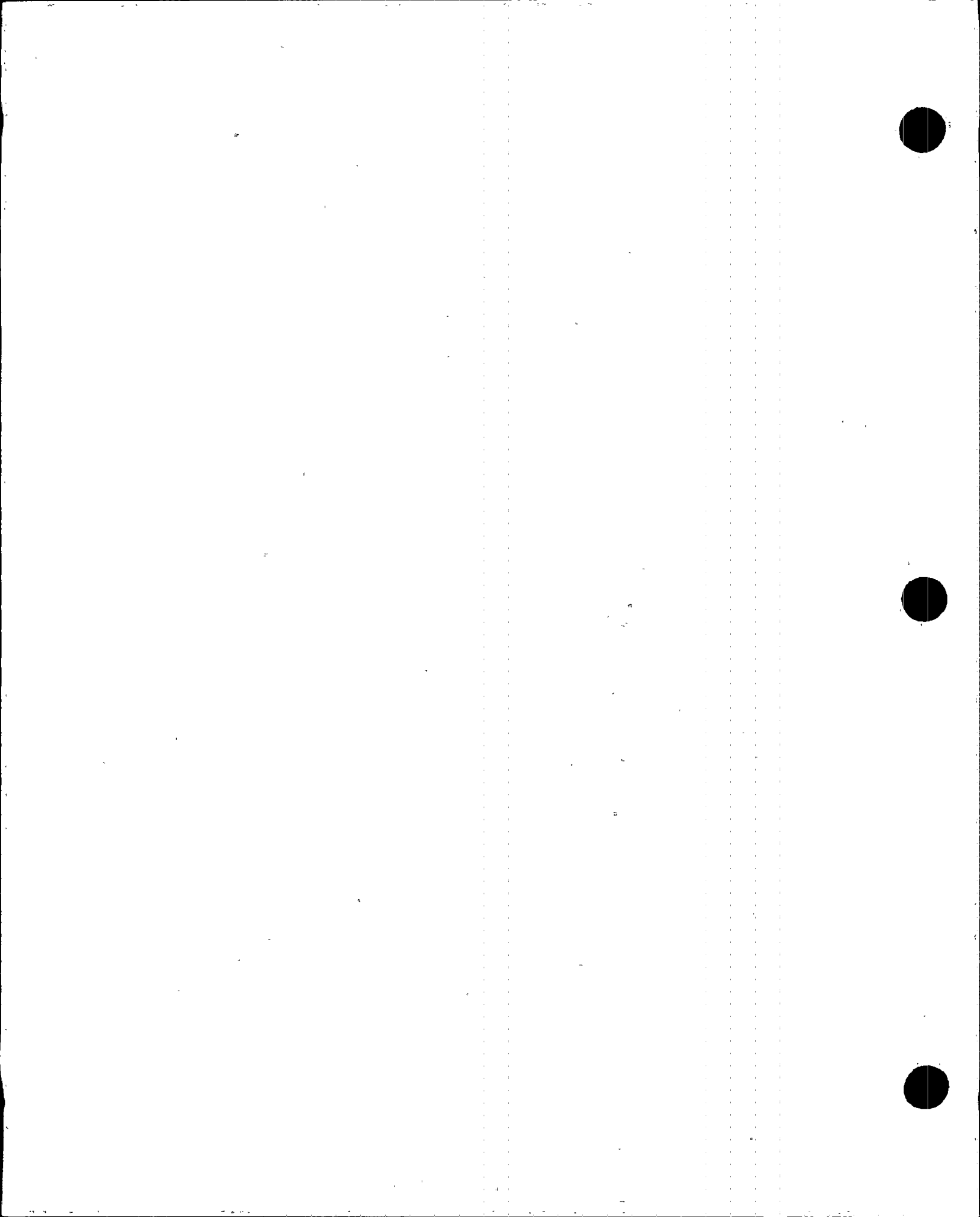
15. Safety Valve Outlets: Number _____ Size _____ Location _____

16 Nozzles	Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached

17 Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Inthreaded, No. _____ Size _____ Location _____

18 Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1. If Packed Head Treated
 2. List other external or internal pressure with supporting formulae when applicable



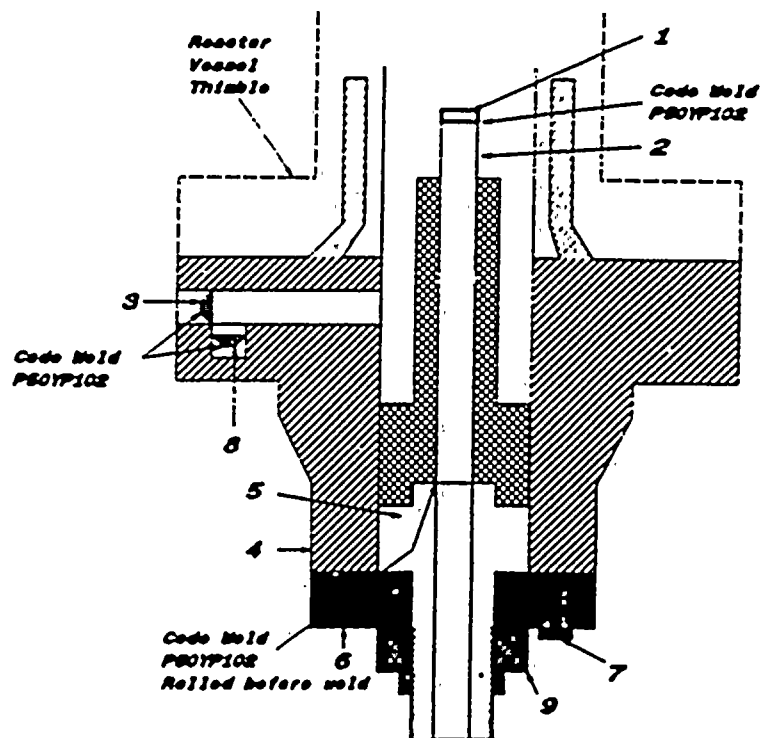
002760

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
 (Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5601 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psf. min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 186B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1178P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 918D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA183 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5834P001
XM - 19 SA479
1.30" thick x 2.82" dia.





002566

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35808-2000
(Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5042 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date, 10/08/96 Signed [Signature] By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MO18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/8/96 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9/96 [Signature] NC 1231, Ohio, WG 3626 PA
Date Inspector's Signature National Board, State, Province And So

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".



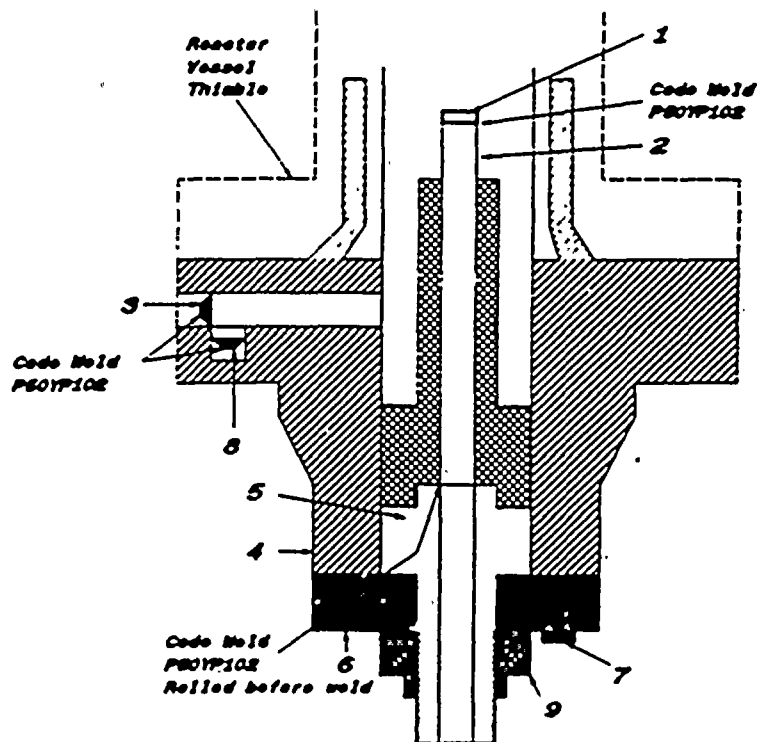
002560

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
 As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
 (Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35809-2000
 (Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5042 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 2 Des. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
 (Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 173A7961P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





002088

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1 Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2200
(Name and Address of R Certificate Holder for completed nuclear component)
- 2 Identification - Certificate Holder's S/N of Part : M298 Part Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
- 3 REMARKS Standard part for use with Reactor Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPI Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report.)

Date: 10/08/96 Signed GE-NE By C. J. Bennett
(NPT Certificate Holder) (QC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. M010646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 7/27/96 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

Date 10/8, 1996 Inspector's Signature Jessie P. Egan National Board, State, Province And No. NC 1231, Ohio, WC 3688 PA

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(97/99)



Items 4-8 Incl to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as open and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____ ft-lb
 Charpy Impact _____ ° F

8. Design pressure ² _____ 1250 psi at _____ 575 ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft.
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____ ft-lb
 Charpy Impact _____ ° F

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Heat, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports. Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat Treated
 2 List either internal or external pressure with coincident temperature when applicable

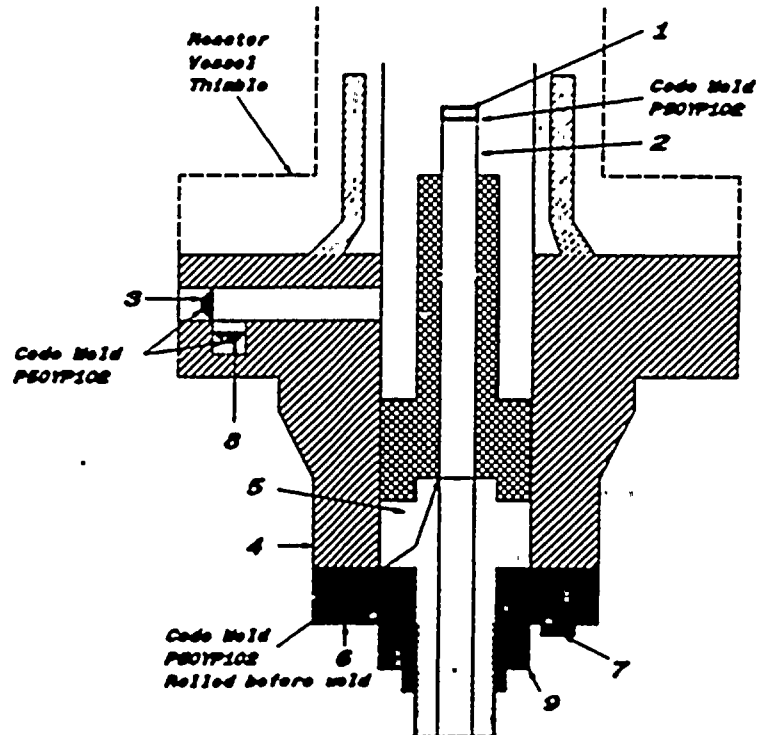


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) G02090
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35608-2000
(Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4288 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B0274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B0313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D810P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C3311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4518P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C3834P001
XM - 19 SA478
1.30" thick x 2.62" dia.





001838

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35602-2000
(Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3758 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1975, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3 REMARKS. Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE-NE By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2

Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1

Stress analysis report certified by Edward Yoshko Prof. Eng. State Calif. Reg. No. AA018648

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 3/10, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996
Date

[Signature]
Inspector's Signature

NC 1231, Ohio, WC 3686 PA
National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/96)



001839

FORM M-2 (back)

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top, Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T & Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe or give dimensions, if bolts, describe or attach)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 incl. to be completed for inner members of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

¹ If Postweld Heat Treated
² Use other internal or external pressure with constant temperature when applicable

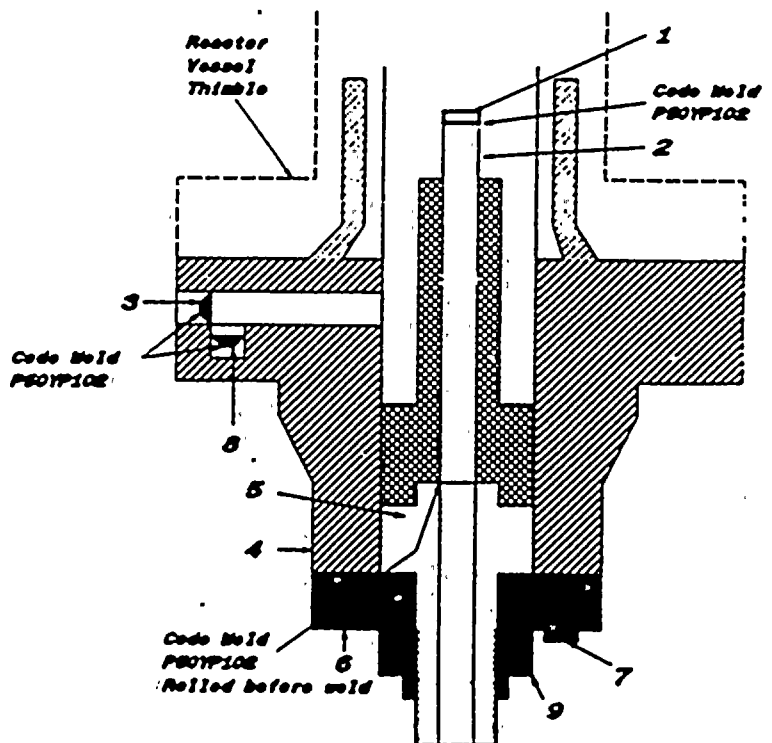


FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) 001840
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A3758 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev. 9, Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # TRDB144EG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi/min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 158A1178P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (71DE474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114BS122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4818P002
SA182 - B8
6 ea. 1/2" dia. on 4 1/8" ball circle
8. Plug 175A7981P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5834P001
XM - 18 SA479
1.30" thick x 2.82" dia.





GC2703

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
 - (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of E Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A5323 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 758E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144EG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 10/08/96 Signed [Signature] By [Signature]
(NPT Certificate Holder) (QC QA Representative)

Certificate of Authorization Expires: 8/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California
Stress analysis report on file at GE Company, San Jose, California
DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345
DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MO18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 5/7, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9, 1996 [Signature] NC 1231, Ohio, WC 3688 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)

002704

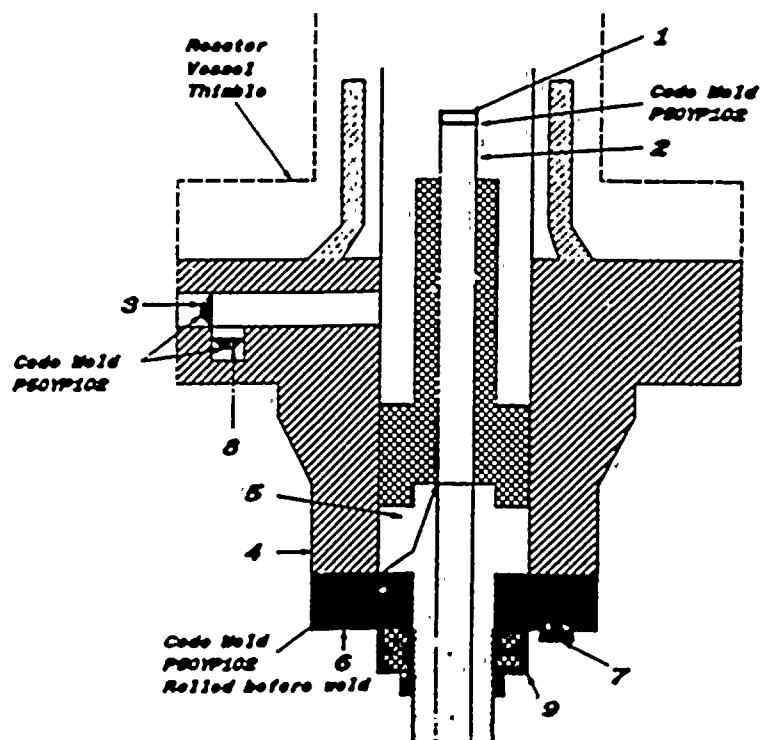
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A5323 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Des. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 158A1178P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 818D610P001 (718E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





002457

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
- 2. Identification - Certificate Holder's S/N of Part : A4809 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 728E534G008 Rev 9 Dwg. Prepared by D.L. Paterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3. REMARKS Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report ;

Date: 10/08/96 Signed _____ By M. Maggett
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif Reg. No. M018646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/9/1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9, 1996 Jessie P. Evers NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/90)



Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as eggs and weld, bar, etc. if bar give dimensions, if bolts, describe or sketch)

Drop Weight _____ ft-lb
 Charpy Impact _____

8. Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____ ft-lb
 Charpy Impact _____

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Manholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat Treated.
 2 List other internal or external pressure with associated temperature when applicable



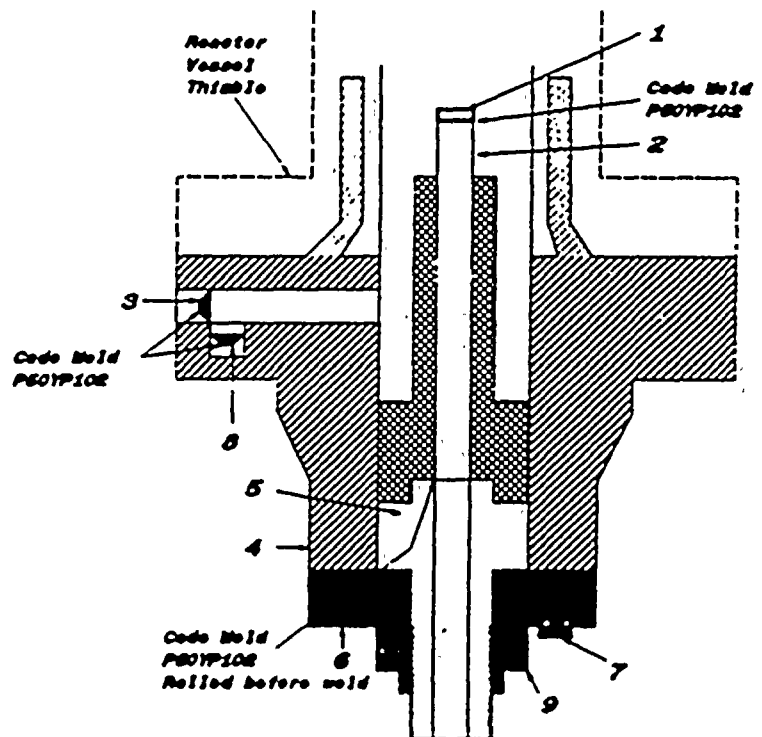
002459

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4809 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D.L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1178P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA182 - B8
8 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 173A7961P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5834P001
XM - 19 SA479
1.30" thick x 2.62" dia.





602316

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
- 2 Identification - Certificate Holder's S/N of Part : A4722 Nat'l Bd. No. N/A
 - (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
 - (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
 - (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
- 3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi. min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report)

Date: 10/08/96 Signed GE By C. B. Smith
(NPT Certificate Holder) (ASME Code Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MO18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 6/28, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection

10/9, 1996 James P. Evers NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(17/90)



FORM M-2 (back)

002317

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if both, describe or sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

8. Design pressure ² _____ 1250 psi at _____ 525 °F at temp of _____ °F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner chambers of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Kind & Spec. No.) (Min. of Flange Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

Drop Weight _____
 Charpy Impact _____ ft-lb

14. Design pressure ² _____ psi at _____ °F at temp of _____ °F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Test, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17. Inspection Manholes, No. _____ Size _____ Location _____
 Openings: Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports. Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat-Treated.
 2 List other internal or external pressure with associated temperature when applicable



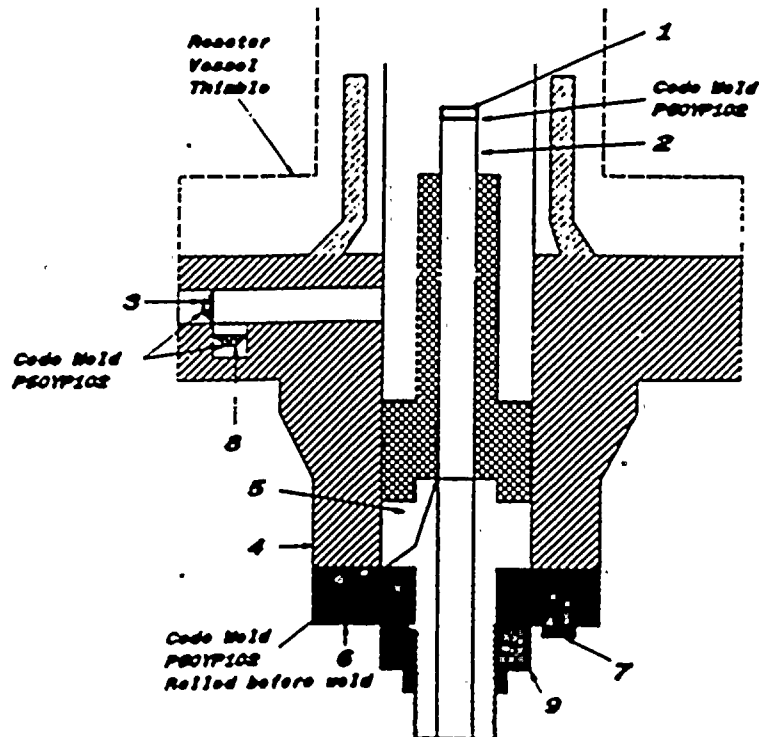
002318

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4722 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2 Class 1
3. REMARKS: Standard part for use with Reactor. Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 158A1176P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D810P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137CS311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117CA516P002
SA193 - B8
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137CS934P001
XM - 19 SA479
1.30" thick x 2.62" dia.







FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

- 1 Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE) **002730**
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35809-2000
(Name and Address of N Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/R of Part : A5524 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 768E534G008 Rev 9 Desg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code, Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1
- 3 REMARKS Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report);

Date: 10/08/96 Signed by C. E. [Signature] By [Signature]
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/16/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A0253 Rev 2
 Design specification certified by B. N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
 Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MO10646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 524, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/9, 1996 [Signature] NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in Item 3. "REMARKS".

(07/96)



FORM M-2 (back)

002731

Items 4-8 incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Code & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location (Top Bottom, Ends)	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) _____	_____	_____	_____	_____	_____	_____	_____	_____
(b) _____	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7 Jacket Closure: _____
(Describe as edge and weld, bar, etc. If bar give dimensions, if bolts, describe or sketch)

8 Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary, Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Code & Spec. No.) (Subject to pressure) (Welded, Bolted)

Floating, Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(Str. or U)

Items 11 - 14 incl. to be completed for inner diameters of jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____ in.
(Code & Spec. No.) (Min. of Range Specified)

12 Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13 Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____

Location	Thickness	Crown Radius	Knuckle Radius	Elliptical Ratio	Conical Apex Angle	Hemispherical Radius	Flat Diameter	Side to Press. (conv. or conc.)
(a) Top, bottom, ends	_____	_____	_____	_____	_____	_____	_____	_____
(b) Channel	_____	_____	_____	_____	_____	_____	_____	_____

If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)

14 Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

Purpose (Inlet, Outlet, Drain)	Number	Dia. or Size	Type	Material	Thickness	Reinforcement Material	How Attached
_____	_____	_____	_____	_____	_____	_____	_____
_____	_____	_____	_____	_____	_____	_____	_____

17 Inspection Manholes, No. _____ Size _____ Location _____
 Openings: Handholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18 Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 If Postweld Heat Treated.
 2 List other internal or external pressure with operating temperature when applicable



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)

002732

3901 Castle Hayne Blvd, Wilmington, North Carolina 28401

(Name and Address of ERT Certificate Holder)

(b) Manufactured for : TVA DECATUR, AL 35608-2000

(Name and Address of E Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A5524 Nat'l Bd. No. N/A

(a) Constructed According to Drawing No: 768E534G008 Rev 9 Dwg. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005

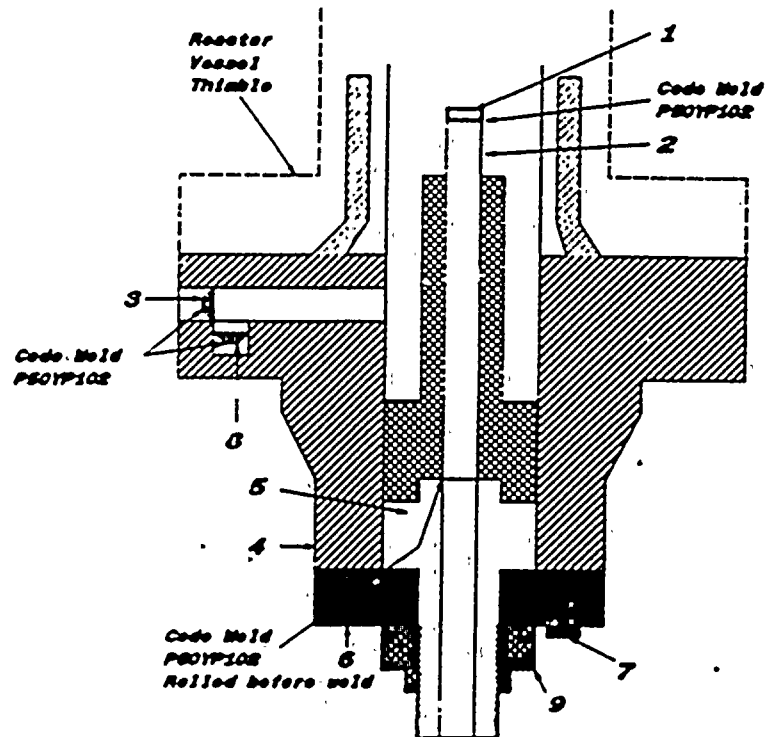
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207 1361-2, Class 1

3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi/min.

(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 166B9274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B9313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 150A1176P001
SA182 - F304
1/4" thick x 0.912" OD
4. Flange 918D610P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
1" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4516P002
SA193 - B6
6 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7961P001
SA182 - F304
0.36" thick x 1.307" dia.
9. Nut 137C5934P001
XM - 18 SA479
1.30" thick x 2.62" dia.





001978

FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES*
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)
3901 Castle Hayne Road, Wilmington, North Carolina 28401
(Name and Address of NPT Certificate Holder)
- (b) Manufactured for : TVA DECATUR, AL 35609-2000
(Name and Address of R Certificate Holder for completed nuclear component)
2. Identification - Certificate Holder's S/N of Part : A4158 Nat'l Bd. No. N/A
- (a) Constructed According to Drawing No: 76BE534G008 Rev 9 Dwg. Prepared by D. L. Peterson
- (b) Description of Part Inspected: Control Rod Drive, Model # 7RDB144FG005
- (c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207-1361-2 Class 1
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi, min.
(Brief description of service for which component was designed)

Sheet 1 of 2

We certify that the statements in this report are correct and this vessel part or appurtenance as defined in the code conforms to the rules of construction of the ASME Code Section III. (The applicable Designed Specification and Stress Report are not the responsibility of the NPT Certificate Holder for parts. An NPT Certification Holder for appurtenances is responsible for furnishing a separate Design Specification and Stress Report if the appurtenance is not included in the component Design Specification and Stress Report).

Date: 10/08/96 Signed GE-NE By C. R. Bazzett
(NPT Certificate Holder) (SC QA Representative)

Certificate of Authorization Expires: 6/18/99 Certification of Authorization No. : NPTN-1151

Certification of Design for Appurtenance

Design information on file at GE Company, San Jose, California

Stress analysis report on file at GE Company, San Jose, California

DC22A6253 Rev. 2
Design specification certified by B.N. Sridhar Prof. Eng. State Calif. Reg. No. 18345

DC22A6254 Rev 1
Stress analysis report certified by Edward Yoshio Prof. Eng. State Calif. Reg. No. MQ18646

Certification of Shop Inspection

I, the undersigned, holding a valid commission by the National Board of Boiler and Pressure Inspectors and/or the State or Province of North Carolina and employed by Department of Labor of State of North Carolina have inspected the part of a pressure vessel described in this Partial Data Report on 10/8, 1996 and state that to the best of my knowledge and belief, the NPT Certificate Holder has constructed this part in accordance with the ASME Code Section III.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the part described in the Partial Data Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damages or a loss of any kind arising from or connected with this inspection.

10/8, 1996 Juan P. Gove NC 1231, Ohio, WC 3686 PA
Date Inspector's Signature National Board, State, Province And No.

*Supplemental sheets in form of lists, sketches or drawing may be used provided (1) size is 8-1/2" x 11", (2) information in 1-2 on this Data Report is included on each sheet, and (3) each sheet is numbered and number of sheets is recorded in item 3. "REMARKS".

(07/00)



Items 4-8 Incl. to be completed for single wall vessels, jackets vessels, or shells of heat exchangers.

4. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Range Specified)

5. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

6. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location (Top Bottom, Ends) Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) _____
 (b) _____
 If removable, bolts used _____ Other fastening _____
(Material, Spec. No., T.S. Size Number) (Describe or attach sketch)

7. Jacket Closure: _____
(Describe as cage and weld, bar, etc. If bar give dimensions, if both, describe or sketch)
 Drop Weight _____ ft-lb
 Charpy Impact _____

8 Design pressure ² _____ 1250 _____ psi at _____ 575 _____ ° F at temp of _____ ° F

Items 9 and 10 to be completed for tube sections

9. Tube Sheets: Stationary. Material _____ Dia. _____ Thickness _____ in. Attachment _____
(Kind & Spec. No.) (Subject to pressure) (Welded, Bolted)
 Floating. Material _____ Dia. _____ Thickness _____ in. Attachment _____

10. Tubes: Material _____ O.D. _____ in. Thickness _____ inches or gage. Number _____ Type _____
(St. or U)

Items 11 - 14 Incl. to be completed for inner chambers in jacketed vessels, or channels of heat exchangers.

11. Shell: Material _____ T.S. _____ Nominal Thickness _____ in. Corrosion Allowance _____ in. Dia. _____ ft. _____ in. Length _____ ft. _____
(Kind & Spec. No.) (Min. of Range Specified)

12. Seams: Long _____ H.T. _____ R.T. _____ Efficiency _____ %
 Girth _____ H.T. _____ R.T. _____ No. of Courses _____

13. Heads: (a) Material _____ T.S. _____ (b) Material _____ T.S. _____
 Location Thickness Crown Radius Knuckle Radius Elliptical Ratio Conical Apex Angle Hemispherical Radius Flat Diameter Side to Press. (conv. or conc.)
 (a) Top, bottom, ends _____
 (b) Channel _____
 If removable, bolts used (a) _____ (b) _____ (c) _____ Other fastening _____
(Describe or attach sketch)
 Drop Weight _____ ft-lb
 Charpy Impact _____

14. Design pressure ² _____ psi at _____ ° F at temp of _____ ° F

Items below to be completed for all vessels where applicable.

15. Safety Valve Outlets: Number _____ Size _____ Location _____

16. Nozzles: Purpose (Inlet, Outlet, Drain) _____ Number _____ Dia. or Size _____ Type _____ Material _____ Thickness _____ Reinforcement Material _____ How Attached _____

17. Inspection Openings: Manholes, No. _____ Size _____ Location _____
 Manholes, No. _____ Size _____ Location _____
 Threaded, No. _____ Size _____ Location _____

18. Supports: Skirt _____ Lugs _____ Legs _____ Other _____ Attached _____
(Yes or No) (Number) (Number) (Describe) (Where & How)

1 - If Postweld Heat Treated.
 2 - List other internal or external pressure with condensing temperature when applicable



FORM N-2 NPT CERTIFICATE HOLDERS' DATA REPORT FOR NUCLEAR PART AND APPURTENANCES
As required by the Provision of the ASME Code Rules, Section III, Div. I

1. Manufactured & Certified by : General Electric Company Nuclear Energy (GE-NE)

001980

3901 Castle Hayne Road, Wilmington, North Carolina 28401

(Name and Address of NPT Certificate Holder)

(b) Manufactured for : TVA DECATUR, AL 35609-2000

(Name and Address of N Certificate Holder for completed nuclear component)

2. Identification - Certificate Holder's S/N of Part : A4159 Part Bd. No. N/A

(a) Constructed According to Drawing No: 788E534G008 Rev. 9 Des. Prepared by D. L. Peterson

(b) Description of Part Inspected: Control Rod Drive Model # 7RDB144FG005

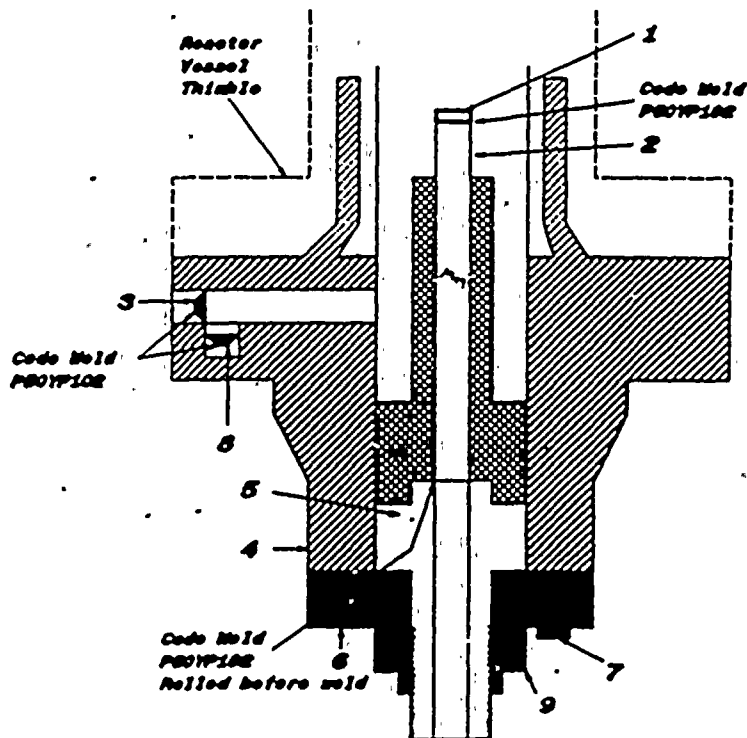
(c) Applicable ASME Code: Section III, Edition 1974, Addenda Date W75, Case No. N207.1391-2, Class 1

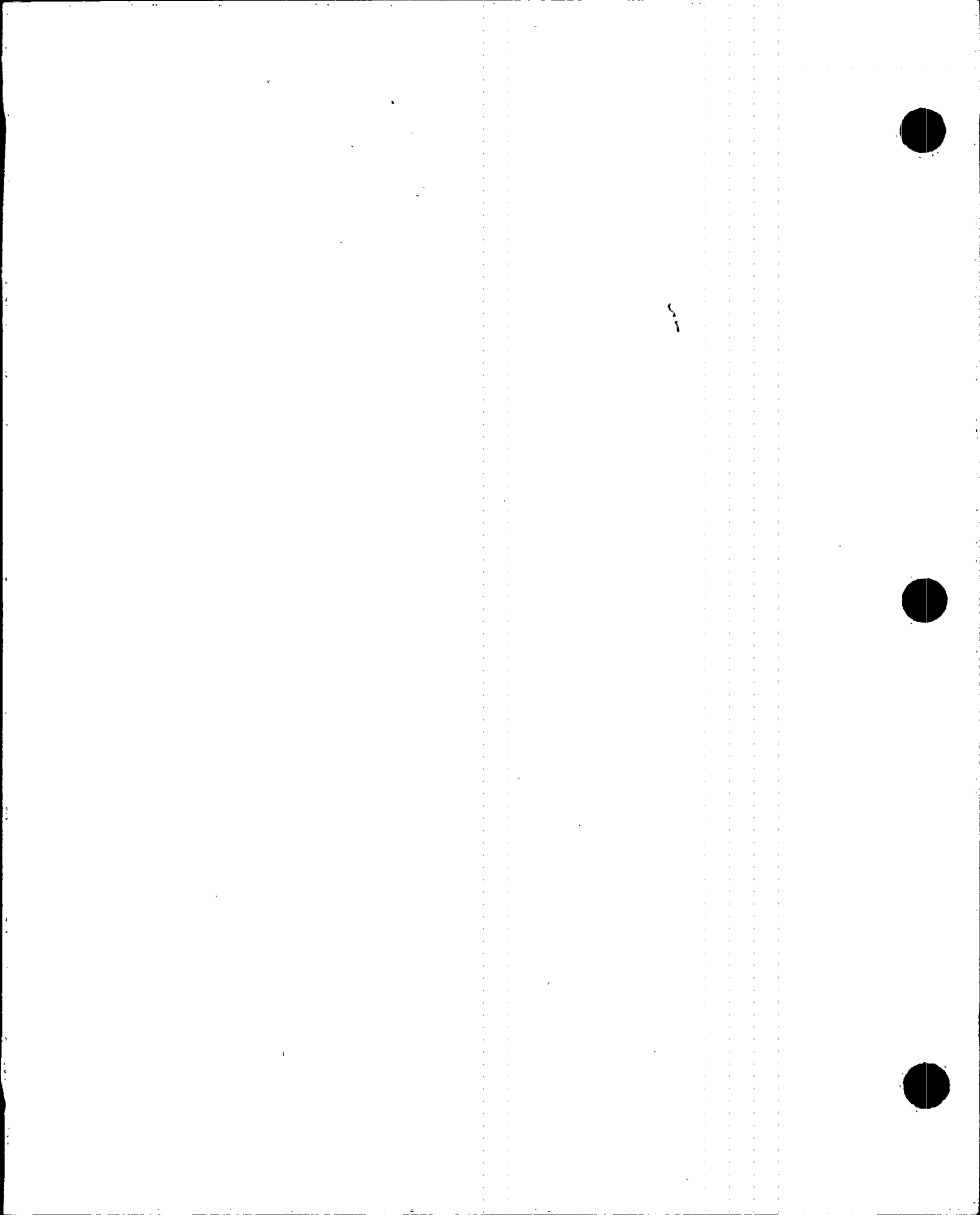
3. REMARKS: Standard part for use with Reactor, Hydrostatically tested at 1825 psi/min.

(Brief description of service for which component was designed)

Sheet 2 of 2

1. Cap 186B8274P001
SA182 - F316
3/8" thick x 1 1/16" OD
2. Indicator Tube 166B0313P001
SA312 - TP316
3/4" sch 40 - seamless pipe
0.113" wall thickness
1.065" max. dia.
3. Plug 159A1178P001
SA182 - F304
1/4" thick x 0.812" OD
4. Flange 919D810P001 (719E474)
SA182 - F304
3.37" thick x 9 5/8" OD
5. Base 137C5311P001
SA182 - F304
7/8" thick x 2.875" dia.
6. Ring Flange 114B5122P002, P003
137C8151P001, P002
SA182 - F304
7" thick x 5.0" OD x 1.75" ID
7. Cap Screw 117C4518P002
SA193 - B8
8 ea. 1/2" dia. on 4 1/8" bolt circle
8. Plug 175A7981P001
SA182 - F304
0.38" thick x 1.307" dia.
9. Nut 137C5634P001
XM-19 SA479
1.30" thick x 2.62" dia.





FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 1, 1999
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Work Order 99-000228-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 075, Core Spray System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Core Spray piping, reducing elbow	Custom Alloy Co.	N/A	N/A	N/A	N/A	Repaired	No

7. Description of Work Ground out indications in the weld metal and blended weld with base metal to a smooth transition.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure Pressure testing is exempted for repairs which do not penetrate the pressure boundary.
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 823005 and Design Criteria BFN-50-7075 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Linear indications were detected in the toe of a weld of an integral attachment using a magnetic particle examination.
The indications were removed by light grinding and the weld metal was blended with base metal to a smooth transition.
The magnetic particle examination following the repair was acceptable.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Stephen C. Williams, System Engineer Date 6-16 19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBIEI of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4-17-99 to 4-22-99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Ladd Commissions TN3135 IB SNA
Inspector's Signature National Board, State, Province, and Endorsements

Date June 18 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 16, 1999

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order 99-000237-000
 Design Change Notice (DCN) S41270A
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 001, Main Steam System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Pipe Whip Restraint Support	Bergen-Paterson	X-7A	N/A	2-47B400S0005	N/A	Replacement	No
Pipe Whip Restraint Support	Bergen-Paterson	X-7B	N/A	2-47B400S0006	N/A	Replacement	No
Pipe Whip Restraint Support	Bergen-Paterson	X-7C	N/A	2-47B400S0007	N/A	Replacement	No
Pipe Whip Restraint Support	Bergen-Paterson	X-7D	N/A	2-47B400S0008	N/A	Replacement	No

7. Description of Work Removed shims welded to integral pipe attachments to allow scheduled inservice inspections on three supports. DCN S41270A allowed the shims to be replaced by bolting them to the support structure. New shim material was used for the replacement shims.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8 1/2 in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91062 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Removed shims welded to integral pipe attachments to allow scheduled inservice inspections on three supports.
Supports 2-47B400S0005, 2-47B400S0006, 2-47B400S0007 and 2-47B400S0008 were disassembled to allow inspection of weld inspections.
Applicable Manufacturer's Data Reports to be attached
DCN S41270A allowed the shims to be replaced by bolting them to the support structure. New shim material was used for the replacement shims.
During disassembly 4 support spacer bolts were damaged on support 2-47B400S0008. All 16 support spacer bolts were replaced on all 4 supports.

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A

Expiration Date N/A

Signed *Stephen C. Williams* System Engineer
Owner or Owner's Designee, Title

Date 6-21, 19 99

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBTEI of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4-9-99 to 5-9-99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd
Inspector's Signature

Commissions TN 3135 IB SWA
National Board, State, Province, and Endorsements

Date June 24 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 16, 1999

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order 99-000512-000
Repair Organization P.O. No., Job No., etc.

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 068, Reactor Water Recirculation System

5. (a) Applicable Construction Code ASME Section III 19 65 Edition, Summer 1965* Addenda, 1332-1, 1332-2, Code Case 1334, 1335-2, 1336

(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Reactor Vessel	Ishikawajima-Harima Heavy Industries Co.	5501-152	N/A	2-RPV-068-1000	1971	See below	No
<i>The fuel support castings are considered part of the core support structures of the reactor vessel</i>							
Fuel Support Casting	General Electric	Unknown	N/A	N/A	N/A	Replaced	No
Fuel Support Casting	General Electric	009	N/A	N/A	N/A	Replacement	No

7. Description of Work Replaced one fuel support casting.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in GE Specification 22A2527 and GE Purchase Order 205-55577.

FORM NIS-2 (Back)

9. Remarks Replaced one fuel support casting due to mechanical fit / dimensional concerns. The fuel support castings are considered
part of the core support structures but were not included on the original FORM N-1 data report for the Unit 2 reactor vessel.
Applicable Manufacturer's Data Reports to be attached
A partial data report is not required or provided with these replacement parts from the vendor (General Electric).

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the
repair or replacement
 ASME Code Section XI.

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Williams*, System Engineer Date 6-16, 19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State
 or Province of TENNESSEE and employed by HSB&E of
HARTFORD, CT have inspected the components described
 in this Owner's Report during the period 4-20-99 to 5-4-99, and state that
 to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's
 Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the
 examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in
 any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd
Inspector's Signature

Commissions TN3135 IB SNA
National Board, State, Province, and Endorsements

Date June 18 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA)
Name
1101 Market Street
Chattanooga, TN 37402-2801
Address

Date June 1, 1999

Sheet 1 of 1

2. Plant Browns Ferry Nuclear Plant (BFN)
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Unit 2

Work Order 99-004328-000
Repair Organization P O No., Job No., etc

3. Work Performed by TVA-BFN
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address

Type Code Symbol Stamp N/A

Authorization No. N/A

Expiration Date N/A

4. Identification of System System 085, Control Rod Drive (CRD) System

5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N/A Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
CRD Hydraulic Control Unit Accumulator	General Electric	A4465	N/A	2-ACC-085-718/1823	1969	Replaced	Yes
CRD Hydraulic Control Unit Accumulator	General Electric	A2481	N/A	2-ACC-085-718/1823	1977	Replacement	Yes

7. Description of Work Replaced one CRD Hydraulic Control Unit accumulator.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 90744 and Design Criteria BFN-50-7085 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Replaced one CRD Hydraulic Control Unit accumulator with a new accumulator, same part number from the same vendor.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed Steph C. Williams, System Engineer Date 6-24, 19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBTEI of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4-7-99 to 5-9-99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd
Inspector's Signature

Commissions TN 3135 TB SVA
National Board, State, Province, and Endorsements

Date June 24 19 99

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS

As Required by the Provisions of the ASME Code Section XI

<p>1. Owner <u>Tennessee Valley Authority (TVA)</u> <small>Name</small> <u>1101 Market Street</u> <u>Chattanooga, TN 37402-2801</u> <small>Address</small></p>	<p>Date <u>June 21, 1999</u></p> <p>Sheet <u>1</u> of <u>1</u></p>
<p>2. Plant <u>Browns Ferry Nuclear Plant (BFN)</u> <small>Name</small> <u>P. O. Box 2000, Decatur, AL 35609-2000</u> <small>Address</small></p>	<p>Unit <u>2</u></p> <p>Design Change Notices (DCN) T32610, Stage 1 & T40617, Stages 3 & 4 <u>Work Order 99-002262-000</u> <small>Repair Organization P.O. No., Job No., etc.</small></p>
<p>3. Work Performed by <u>TVA-BFN</u> <small>Name</small> <u>P. O. Box 2000, Decatur, AL 35609-2000</u> <small>Address</small></p>	<p>Type Code Symbol Stamp <u>N/A</u></p> <p>Authorization No. <u>N/A</u></p> <p>Expiration Date <u>N/A</u></p>

4. Identification of System System 069, Reactor Water Cleanup (RWCU) System

5. (a) Applicable Construction Code (isolation valves) ASME Section III, 1989 Edition (piping) USAS B31.1.0 19 67 Edition, N/A Addenda, N-416-1 Code Case 660 7/12/99
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
RWCU Inboard Suction Isolation Valve	Anchor/Darling 6" - 900# DD Gate Vlv	E-471A-5-1	N/A	2-FCV-069-001	1998	Replacement	No
RWCU Outboard Suction Isolation Valve	Anchor/Darling 6" - 900# DD Gate Vlv	E-471A-2-1	N/A	2-FCV-069-002	1998	Replacement	No
RWCU Chemical Decon Shutoff Valve	Vogt 2" A182F316L	1-216919	N/A	2-SHV-069-0551	N/A	Replacement	No
RWCU Chemical Decon Shutoff Valve	Vogt 2" A182F316L	2-216919	N/A	2-SHV-069-0552	N/A	Replacement	No
pipng	TVA	N/A	N/A	N/A	1999	Replacement	No

7. Description of Work Replaced the inboard and outboard RWCU suction line isolation valves with a new design. Added a 2" decontamination connection and isolation valves.

8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contracts 1241382 Supp 1 & 81262E and Design Criteria BFN-50-7069 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Replaced the inboard and outboard RWCU suction line isolation valves with a new design. Added a 2" decontamination connection and isolation valves.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Williams* System Engineer Date 6-24 .19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBT & I of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4-7-99 to 5-9-99 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Todd Commissions TN13135 I B SNA
Inspector's Signature National Board, State, Province, and Endorsements

Date June 25 19 99 WORK PLANS T40617-005, T40617-006 & T32610-001

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 21, 1999
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
Address
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Design Change Notice (DCN) T40713A, Stage 4 and
Work Order (WO) 99-003061-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 071, Reactor Core Isolation Cooling (RCIC) System
5. (a) Applicable Construction Code (valve) ASME, Section III, 1995 Edition, Winter 1997 Addenda
(piping) USAS B31.1.0 19 67 Edition, N/A Addenda, N-416-1 Code Case
- (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
RCIC Steam Line Isolation Valve	Anchor/Darling	E488A-1-1	N/A	2-FCV-071-0002	1998	Replacement	No
piping	TVA	N/A	N/A	N/A	1999	Replaced	No

7. Description of Work Replaced the RCIC steam supply line inboard isolation valve and some associated piping.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 1240323 Supp 4 and Design Criteria BFN-50-7071 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Replaced the RCIC steam supply line inboard isolation valve and some associated piping.
Applicable Manufacturers Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed [Signature] System Engineer Date 6-24 1999
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of TENNESSEE and employed by HSBZZ of HARTFORD, CT have inspected the components described in this Owner's Report during the period 4-7-99 to 5-9-99, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions IN 3135 T B SVA
Inspector's Signature National Board, State, Province, and Endorsements

Date June 25 1999 WORK PLAN T 40713-003

FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS
As Required by the Provisions of the ASME Code Section XI

1. Owner Tennessee Valley Authority (TVA) Date June 21, 1999
Name
1101 Market Street
Address
Chattanooga, TN 37402-2801
2. Plant Browns Ferry Nuclear Plant (BFN) Unit 2
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
Work Order 99-005552-000
Repair Organization P.O. No., Job No., etc.
3. Work Performed by TVA-BFN Type Code Symbol Stamp N/A
Name
P. O. Box 2000, Decatur, AL 35609-2000
Address
 Authorization No. N/A
 Expiration Date N/A
4. Identification of System System 001, Main Steam (MS) System.
5. (a) Applicable Construction Code USAS B31.1.0 19 67 Edition, N/A Addenda, N-416-1 Code Case
 (b) Applicable Edition of Section XI Utilized for Repairs or Replacements 19 89

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
MS piping to Off-Gas Recombiner	Dravo	N/A	N/A	N/A	N/A	Replaced	No
MS piping to Off-Gas Recombiner	Capitol Mfg. Co. (tee & reducing fitting) U. S. Steel Group (pipe)	N/A	N/A	N/A	N/A	Replacement	No

7. Description of Work Replaced two segments of 2" pipe, a 2" pipe tee and a 2"x3/4" reducing fitting.
8. Tests Conducted: Hydrostatic Pneumatic Nominal Operating Pressure
 Other Pressure N/A psi Test Temp. N/A °F

NOTE: Supplemental Sheets in form of lists, sketches, or drawings may be used, provided (1) size is 8½ in. X 11 in., (2) information in items 1 through 6 on this report is included on each sheet, and (3) each sheet is numbered and the number of sheets is recorded at the top of this form.

*as amended by additional quality assurance requirements found in Contract 68C37-91062 and Design Criteria BFN-50-7001 and BFN-50-C-7105.

FORM NIS-2 (Back)

9. Remarks Replaced two segments of 2" pipe, a 2" pipe tee and a 2"x3/4" reducing fitting.
Applicable Manufacturer's Data Reports to be attached

CERTIFICATE OF COMPLIANCE

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code Section XI.
repair or replacement

Type Code Symbol Stamp N/A

Certificate of Authorization No. N/A Expiration Date N/A

Signed *Stephen C. Williams* System Engineer Date 7-2 19 99
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of TENNESSEE and employed by HSBTEI of HARTFORD, CT have inspected the components described in this Owner's Report during the period 5-7-99 to 7-3-99 and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Albert Lull
Inspector's Signature

Commissions TN3135 IBSNA
National Board, State, Province, and Endorsements

Date July 3 19 99